

LICENSEE EVENT REPORT (LER)

Facility Name (1) Braidwood, Unit 2 Docket Number (2) 0 5 0 0 0 4 5 7 Page (3) 1 of 0 3

Title (4) Missed Technical Specification Surveillance Due to Computational Error

Event Date			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0 6	2 9	8 8	8 8	0 1 1 7	0 0	0 7	2 0	8 8	NONE	0 5 0 0 0 1 1

OPERATING MODE (9) 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

POWER LEVEL (10) <u>0 4 9</u>	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	Other (Specify in Abstract below and in Text)
	20.405(a)(1)(iii) <u>X</u>	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name Dennis Robison, Station Surveillance Coordinator, ext. 2662 TELEPHONE NUMBER 8 1 5 4 5 6 - 2 8 0 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

Expected Submission Date (15) X YES (If yes, complete EXPECTED SUBMISSION DATE) NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1500 on June 29, 1988, a routine review of completed surveillance 2Bw05 4.6.2.1.D-1, Reactor Coolant System Water Inventory Balance 72 Hour Surveillance, performed June 2, 1988, was completed. It was discovered that the surveillance was invalid as a result of calculational errors which did not reflect that the unidentified leakage was actually a negative amount larger than the error factor which requires the surveillance to be repeated. This event was a result of a calculation error by the Nuclear Station Operator, Licensed Reactor Operator. This event has been reviewed with the personnel involved. Emphasis was given to the importance of attention to detail in the performance of this surveillance. Also stressed was the absolute necessity of adequate reviews to minimize the possibility of errors going undetected.

There have been no previous occurrences.

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TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [xx]								

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: Braidwood 2; Event Date: June 2, 1988; Event Time: 0000
 MODE: 1 - Power Operation; Rx Power: 49%; RCS [AB] Temperature/Pressure: 570 Degrees F/2235 psig

B. DESCRIPTION OF EVENT:

There were no systems or components inoperable at the beginning of the event which contributed to the severity of the event.

At 1500 on June 29, 1988, a routine review of completed surveillance package 2BwOS 4.6.2.1.D-1, Unit Two - Reactor Coolant System Water Inventory Balance 72 Hour Surveillance, performed on June 2, 1988, at 0000 was completed. It was discovered that the surveillance was invalid as a result of calculational errors which did not reflect that the unidentified leakage was actually a negative amount larger than the error factor which requires the surveillance to be repeated. As a result, of this surveillance being invalid, there was no valid surveillance of RCS unidentified leakage during a time interval of 109 hours and 24 minutes between 0400 on May 30, 1988 and 1724 on June 3, 1988. This exceeds the allowable absolute time limit of 90 hours for performance of this surveillance (72 hours times 1.25 = 90 hours) by 19 hours and 24 minutes.

The combine. time interval for any three consecutive surveillances of RCS Unidentified Leakage was checked and verified ... be within the specified 3.25 times the specified surveillance interval of 72 hours or 234 hours.

A review of the RCS Unidentified Leakage Surveillances conducted prior to and subsequent to the 0000 June 2, 1988 surveillance, indicated no calculational errors and that leakage was within allowable limits at all times.

Operator actions neither increased nor decreased the severity of the event. Plant conditions were unaffected by this event.

This event is reportable pursuant to 10CFR50.73(a)(2)(i) - any operation or condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT:

This event was a result of a calculation error by the Nuclear Station Operator, Licensed Reactor Operator. There were no adverse environmental conditions or unusual activity at the time of the error. The calculational error was not discovered during Operating Shift Review by two shift Licensed Senior Reactor Operators.

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D. SAFETY ANALYSIS:

There was no affect on plant or public safety as the RCS unidentified leakage was verified to be within allowable limits for the surveillances performed prior to an subsequent to June 2, 1988. Under worst case conditions of leakage in excess of the Technical Specification limits and High RCS Radioactivity, there would have been no affect on plant or public safety as the other leakage detection systems were operable.

E. CORRECTIVE ACTIONS:

This event has been reviewed with the personnel involved. Emphasis was given to the importance of attention to detail in the performance of this surveillance. Also stressed was the absolute necessity of adequate reviews to minimize the possibility of errors going undetected.

F. PREVIOUS OCCURRENCES:

No previous occurrences of missed technical specification surveillances due to calculational errors.

G. COMPONENT FAILURE DATA:

This event was not caused by equipment failure, nor did any equipment fail as a result.



Commonwealth Edison
Braidwood Nuclear Power Station
Route #1, Box 84
Braceville, Illinois 60407
Telephone 815/458-2801

BW/88-813

July 21, 1988

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Braidwood Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2) (i) which requires a 30 day written report.

This report is number 88-017-00; Docket No. 50-457.

Very truly yours,

R. E. Querio
Station Manager
Braidwood Nuclear Station

REQ/PMB/jab
(7126z)

Enclosure: Licensee Event Report No. 88-017-00

cc: NRC Region III Administrator
NRC Resident Inspector
INPO Record Center
CECo Distribution List

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