AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	Fort Calhoun Station
DATE	April 12, 1988
COMPLETED BY	W. J. Blessie
TELEPHONE	402-536-4595

JEZ4

1/1

MONTH_	March 1988		
UAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	332.4	17 _	330.8
2	332.1	18 _	331.2
3 _	332.3	19 _	331.0
4	331.8	20 _	330.4
5	332.6	21 _	329.9
6	333.0	22 _	329.8
7	332.9	23 _	329.1
8	332.3	24 _	329.0
9	331.8	25 _	330.0
10	331.8	26 _	330.4
11	331.1	27 _	330.3
12	331.2	28 _	330.4
13	331.9	29 _	330.5
14	332.0	30 _	330.9
15	331.9	31 _	331.2
16	330.2		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

8804200142 880331 PDR ADOCK 05000285 R DCD

DOCKET NO.	50-285
UNIT	Fort Calhoun Station
DATE	April 12. 1988
COMPLETED BY	W. J. Blessie
TELEPHONE	402-536-4595

OPERATING STATUS

Maximum Dependable Capacity (Gross MWe): Maximum Dependable Capacity (Net MWe): If changes occur in Capacity Ratings (It Give Reasons: N/A	: <u>502</u> 478 tems Number 3	Through 7) Sind	ce Last Rep
Power Level to Which Restricted, If Any Reasons for Restrictions, If Any:	(Net MWe):	N/A	
	This Month	Yr-to-Date	Cumulati
Hours in Reporting Period	744.0	2184.0	127.2
Number of Hours Reactor Was Critical		2184.0	99.0
Hours Generator On-Line	744.0	2184.0	98,1
Unit Reserve Shutdown Hours	0.0	0.0	107 067 6
Gross Thermal Energy Generated (MWH)	782.732.2	924 842 0	41 897 7
Net Electrical Energy Generated (MWH)	246.382.8	876,573.8	40,020,6
Unit Service Factor	100.0	100.0	
Unit Availability Factor	100.0	100.0	
Jnit Capacity Factor (Using MUC Net)	69.3	84.0	
Init Forced Outage Rate	0.0	0.0	
Shutdowns Scheduled Over Next 6 Months (Type, Date, an	d Duration of	Each):
The 1988 Refueling Shutdown is tentati	vely scheduled	for September	2, 1988,
startup tentatively scheduled for Novemb	timated Date of	f Startun N	1/4
IT Shut Down at the of Report Period, ts	1 One time to	Foundation	Achiound

UNIT SHUTDOWNS AND POWER REDUCTIONS

UNIT NAME Fort Calhoun Station DATE April 12, 1988 COMPLETED BY W. J. Blessie TELEPHONE 402-536-4595

REPORT MONTH March 1988

and the lot of the lot									
N	Date	Typel	Duration (flours)	Reason?	Method of Shutting own Reactor 3	Licensee Event Report =	System Cude ⁴	Component Cude ⁵	Cause & Corrective Action to Prevent Recurrence
38-01	880212	S	0	Н	4	N/A	ВК	FAN	On February 12, 1988, a power reduc- tion commenced to repair a nuclear detector well cooling fan in contain- ment. Power was hekd at 35% while repairs were made and power was then returned to 70% on February 15,1988. Power will remain at 70% for approx- imately seven weeks to extend the fuel burnup window to the desired shutdown date in September 1988.
1 F Fe S Sc (9/77)	nccd heduled	Reas A-Eo B-Mi C-Re D-Re E-Oj F-Ao G-Oj H-O	on puipment Fa intenance of fueling gulatory R berator Trai Immistrativ perational E ther (Explai	ailure (E or Test estrictio ning & 1 e trror (Eo in)	xplain) n License Exa xplain)	mination	Methoo I-Mani 2-Mani 3-Auto 4-Othe	d: ial aal Scram. imatic Scram. r (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit 1 - Same Source

Refueling Information Fort Calhoun - Unit No. 1

	Report for the month ending	March 1988 .	
1.	Scheduled date for next refuel	ing shutdown	September 1988
2.	Scheduled date for restart fol	lowing refueling.	December 1988
3.	Will refueling or resumption o require a technical specificat license amendment?	of operation thereafter ion change or other —	Yes
	a. If answer is yes, what, i these be?	n general, will	
	Incorporate cycle specifi resulting from reload saf	c requirements ety analysis.	
	b. If answer is no, has the and core configuration be your Plant Safety Review mine whether any unreview are associated with the c	reload fuel design en reviewed by Committee to deter- ed safety questions ore reload.	
	c. If no such review has tak it scheduled?	en place, when is	
i.,	Scheduled date(s) for submitti action and support information	ng proposed licensing •	July 1988
5.	Important licensing considerat refueling, e.g., new or differ supplier, unreviewed design or methods, significant changes i operating procedures.	ions associated with ent fucl design or performance analysis n fuel design, new	
5.	The number of fuel assemblies:	 a) in the core b) in the spent fuel pool c) spent fuel pool storage capacity d) planned spent fuel pool storage capacity 	<u>133</u> assemblie <u>393</u> " May be increased " <u>via fuel pin</u> " consolidation
	The projected date of the last discharged to the spent fuel po licensed capacity.	refueling that can be ool assuming the present	1996
Pre	pared by the shittle	DateMarch	25, 1988

OMAHA PUBLIC POWER DISTRICT Fort Calhoun Station Unit No. 1

March 1988 Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 70% power throughout March 1988 to conserve core reactivity in order to maintain 100% power through the summer and still attain the scheduled September shutdown for refueling. Construction continued on the new training facility, warehouse and maintenance shop.

An MRC inspection was performed on the surveillance test program. A SSOMI team inspected modification package preparation. A training exit was held after administration of NRC license requalification exams. Two new RO licenses and a SRO upgrade license were effective March 14, 1988.

No safety valves or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure Description

SP-STROKE-1 In Service Testing of Air Operated, CQE Valves.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it only allowed stroke testing on HCV-805C as post-maintenance testing. The objective of the testing is to verify valve operability following blowdown of valve internals to remove moisture per mO 880745. This testing did not in any way compromise plant safety, but enhances it by ensuring operability of HCV-805C. Monthly Operations Report March 1988
 Page Two

> D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

SP-CTPC-1

Core Thermal Power Calculation - NSSS Calorimetric.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure is a hand calculation to check the validity of the calculations performed by the plant computer. The results of this check showed that the plant computer was performing as designed in performing the calculation.

System Acceptance Committee Packages for March 1988:

Package Description/Analysis

EEAR FC-81-092 Emergency Exits.

This modification provided for the installation of stairs and a masonary block structure between Room 20 and Room 57, and relocated wiring, conduit and halon piping as required. This modification does not have an adverse effect on the safety analysis.

EEAR FC-85-121 Install Eye Wash/Safety Shower - Water Plant & Turbine Building.

> This modification provided for three new eye wash stations and two new safety showers in the turbine building and water plant. This modification does not have an adverse effect on the safety analysis.

EEAR FC-85-195 Installation of New Environmental Air Sampler.

This modification provided for the installation of the environmental air sampler RM-038 at the Blair OPPD Office. This modification does not have an adverse effect on the safety analysis.

EEAR FC-87-013 Alert Notification System.

This modification provided for an upgrade of the Fort Calhoun Station siren system (3 new siren sites). This modification does not have an adverse effect on the safety analysis. Monthly Operations Report March 1988 Page Three

> D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

System Acceptance Committee Packages for March 1988:

Package Description/Analysis

EEAR FC-87-024 HCV-385 and HCV-386 Air Accumulators.

This modification provided for separate air accumulators for valves HCV-385 and HCV-386. This modification does not have an adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

None

F. CHANGES IN PLANT OPERATING STAFF

During March, Dennis Bonsall was promoted to Licensed Senior Operator, Tony Hargan was promoted to Licensed Operator and Gene Creamer was promoted to Licensed Operator.

G. TRAINING

During March, the Equipment Operator-Nuclear Course was completed by five individuals and the Technical Staff Course was completed by seven individuals. The licensed operator annual requalification examination was administered by OPPD to the majority of licensed operators. The NRC administered the licensed operator requalification examination to eight individuals as part of the requalification examination "Pilot" process. Annual simulator training was begun in licensed operator requalification training.

Maintenance, Chemistry, Health Physics and General Employee Training continued. The Health Physics training is being scheduled to support recent violation responses in this area.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

Monthly Operations Report March 1988 Page Four

II. MAINTENANCE (Significant Safety Related)

None

M. Gary Gates Manager-Fort Calhoun Station

Omaha Public Power District 1623 Harney Omaha. Nebraska 68102-2247 402/536-4000

April 13, 1988 LIC-88-269

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: March Monthly Operating Report

Pursuant to Technical Specification Section 5.9.1, and 10 CFR Part 50.4(b)(1), please find enclosed, one copy of the March 1988 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

Ne anareno

R. L. Andrews Division Manager Nuclear Production

RLA/me

Enclosures

c: NRC Regional Office Office of Management & Program Analysis (2) R. M. Caruso - Combustion Engineering R. J. Simon - Westinghouse Nuclear Safety Analysis Center INPO Records Center American Nuclear Insurers NRC File (FCS)