

OPERATIONS SUMMARY
JUNE 1988

The unit entered June at 100% power. On June 9, T_{ave} reductions were commenced in step reductions of 0.5°F to provide additional core reactivity as observed by 2% rod index changes. Final T_{ave} of 575°F was achieved on June 12. The AFSRs were withdrawn on June 13 to provide additional core reactivity. A controlled reactor shutdown was completed on June 17 to enter the Cycle 7 refueling outage. As a result of a Low Pressure Injection Test conducted per Technical Specifications during the cooldown sequence, LER 88-002 was generated. At the close of the month, TMI-1 was on the decay heat removal cooling system with RCS level in a partially drained down condition to support the outage work scope.

MAJOR SAFETY RELATED MAINTENANCE

During June, TMI-1 performed the following major maintenance activities on safety-related equipment:

Fire Service Pump Diesel FS-P-1 - Repairs to FS-P-1 diesel engine were completed in June to correct an overheating problem. The engine coolant heater check valve was found to be defective and a new heater with check valve was installed. The engine was tested satisfactorily and returned to service.

Spent Fuel Bridge FH-A-3 - Repairs to the Spent Fuel Bridge were accomplished during June. Work activities included replacing a defective Fuel Mast Hose Reel and repairing a water leak at a threaded fitting with Belzona compound. The Bridge was tested and returned to service.

Nuclear Service Closed Cooling Valve NS-V-10A - Check Valve NS-V-10A was opened for inspection. The internals of the valve were found to be loose and required minor repair work. While the valve was open, a disc stop was manufactured and installed per approved Engineering guidance and the disc nut was tack welded. The valve was reassembled, tested and returned to service.

Emergency Reactor River Water Valve RR-V-7A - Check Valve RR-V-7A was opened for inspection. The inspection revealed no deficiencies of the valve internals. The valve was reassembled, tested, and returned to service.

Building Spray Valve BS-V-52B - Check Valve BS-V-52B was opened for inspection and repair of a cover gasket leak. A scheduled IST inspection was also performed. Inspection of the valve internals revealed no deficiencies and the valve was reassembled, tested and returned to service.

Main Steam Safety Valve Testing - With the plant at power, six Main Steam Safety Valves were tested by a vendor representative. Five valves were tested per surveillance 1303-11.3 and one valve tested for "Information Only." Testing revealed that two valves, MS-V-20D and MS-V-21B, would require corrective maintenance. With the plant in cold shutdown, MS-V-20D and MS-V-21B were disassembled. MS-V-21B required lapping to restore the valve seating surfaces. Valve MS-V-20D required replacing the valve disc, lapping seating surfaces and lapping the guide and guide body to correct an out of round condition. The valves were reassembled and will be tested when the system is returned to service. Valve MS-V-21A is to be disassembled for inspection in July.

Once Through Steam Generators RC-H-1A/B - Once Through Steam Generator inspection and repair activities commenced in June. Work preparation activities included removal of the upper primary manways and hand holes, installing ventilation systems with charcoal filters and erecting tents at the upper manways. A bubble test was then performed with no tube leakage evident. After the bubble test, the OTSGs were drained and the lower primary manways and handholes were removed. Tents were then installed at the lower primary manways. Eddy Current Test (ECT) equipment was set up and testing started. After the initial ECT data was reviewed, an additional 6% of the tubes in each OTSG were required to be tested. Preparations for OTSG secondary side mechanical cleaning commenced with removal of "A" and "B" OTSG lower secondary manway and hand holes and the "A" OTSG inspection ports. OTSG mechanical cleaning will start in July.

Reactor Vessel Head Removal Preparation - Preparations for Reactor Vessel Head Removal commenced in June with removal of the missile shields, disconnecting "A" to "B" cables, disconnecting PI/TC cables, removing inventory tracking cables and removal of the reactor head fans. The APSRs and CRDMs were uncoupled with the exception of CRDM N-4. Work will continue in July.

Reactor Coolant Pump Inspections - Reactor Coolant Pump inspection work on RC-P-1 A/B/C/D started in June with scaffold installation, decon work, removal of the oil drip pans, thermocouple and instrumentation removal and installing seal removal hardware. The pump spool pieces and couplings were removed and the seals removed from the seal cavity. Final seal inspections and pump reassembly will be completed in July.

Local Leak Rate Testing (LLRT) - Local Leak Rate Testing (LLRT) started in June. Twenty-seven valves were tested, of which three have higher than target value leakage. Test data of these valves is being reviewed. Testing will continue in July with seventy-one valves remaining to be tested.

REFUELING INFORMATION REQUEST

1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
2. Scheduled date for next refueling shutdown: January 5, 1990 (8R)
3. Scheduled date for restart following refueling: August 19, 1988 (for current 7R Outage) - March 1, 1990 (8R)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

If answer is yes, in general, what will these be?

Basic Refueling Report.

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?
To be determined.

5. Scheduled date(s) for submitting proposed licensing action and supporting information: October 1, 1989 (estimate).
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: None
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 284
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. Preliminary planning to increase licensed capacity through fuel pool reracking is in process.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1991 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE 6-30-88
 COMPLETED BY C.W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: JUNE ,1988.
- 3. LICENSED THERMAL POWER (MWT): 2535.
- 4. NAMEPLATE RATING (GROSS MWE): 871.
- 5. DESIGN ELECTRICAL RATING (NET MWE): 819.
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 824.
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 776.

NOTES

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)

10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	720.	4367.	121224.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	406.0	3991.4	50511.9
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	61.6	1947.7
14. HOURS GENERATOR ON-LINE	405.5	3988.9	49588.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1021504.	10064153.	120551853.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	345657.	3454503.	40322724.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	324347.	3261030.	37765312.
19. UNIT SERVICE FACTOR	56.3	91.3	40.9
20. UNIT AVAILABILITY FACTOR	56.3	91.3	40.9
21. UNIT CAPACITY FACTOR (USING MDC NET)	58.1	96.2	39.9
22. UNIT CAPACITY FACTOR (USING DER NET)	55.0	91.2	38.0
23. UNIT FORCED OUTAGE RATE	0.0	1.6	54.6

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)

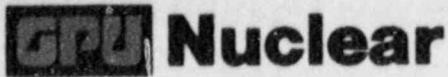
25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 8/19/88

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE 6-30-88
 COMPLETED BY C.W. Smyth
 TELEPHONE (717) 948-8551

MONTH: JUNE

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	794.	17	678.
2	816.	18	-27.
3	822.	19	-11.
4	823.	20	-6.
5	811.	21	-6.
6	804.	22	-6.
7	801.	23	-5.
8	818.	24	-5.
9	822.	25	-5.
10	822.	26	-5.
11	817.	27	-5.
12	807.	28	-5.
13	801.	29	-5.
14	791.	30	-5.
15	791.	31	N/A
16	790.		



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TELEX 84-2386
Writer's Direct Dial Number:

July 15, 1988
C311-88-2091

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report
June 1988

Enclosed please find two (2) copies of the June 1988 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

H. D. Hukill
Vice President & Director, TMI-1

HDH/SMO:spb

cc: W. Russell, USNRC
R. Conte, USNRC

Attachments

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