



Commonwealth Edison
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July 14, 1988

Mr. Thomas E. Murley
Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

Subject: Byron Station Unit 2
Application for Amendment to Facility
Operating License NPF-66
Appendix A, Technical Specifications
NRC Docket No. 50-455

Reference (a): June 22, 1988 K.A. Ainger letter to T.E. Murley

Dear Mr. Murley:

Pursuant to 10 CFR 50.90, reference (a) provided Commonwealth Edison's application for an amendment to Facility Operating Licenses NPF-37 and NPF-66, Appendix A, Technical Specifications. The purpose of this amendment is to revise a technical specification action statement concerning the ultimate heat sink. The particular action statement involves Rock River water level and flow. Attachment A to reference (a) contains the proposed change to action requirement 3.7.5.e.(2). Attachment B to reference (a) contains background information regarding the proposed change.

The purpose of this letter is to request that this amendment be processed under the emergency provisions of 10CFR 50.91a(5) for Facility Operating License NPF-66.

At approximately 1:14a.m. on Thursday, July 14, 1988, Byron Unit 2 experienced an unexpected shutdown due to the inadvertent opening of the two feeder breakers that connect the Unit Aux and System Aux Transformers to the Unit 2 non-safety related bus, which

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resulted in the loss of the 2C feedwater pump. The unit is currently in Mode 3. As stated in Attachment B to reference (a), the ongoing drought in the mid-western part of the nation has caused the level in the Rock River to drop below the limit specified in the limiting condition for operation of the ultimate heat sink technical specification. That limit is 670.6 feet mean sea level (MSL). As a result, Byron Station has entered technical specification action requirement 3.7.5.e. This action requirement permits reactor operation to continue for an unlimited period of time as long as river flow remains greater than 700 cubic feet per second (cfs) and river level remains above 664.7 feet MSL. Since entering the action requirement, the river level has fluctuated between 670 feet - 3 inches and 670 feet - 8 inches and river flow has fluctuated between approximately 1250 cfs and 2100 cfs. Therefore, substantial margin to the technical specification minimum values currently exists. River flow must be verified every 12 hours under these conditions until the level exceeds 670.6 feet MSL.

General technical specification 3.0.4 applies to technical specification 3.7.5. According to specification 3.0.4, entry into an operational mode or other specified condition cannot be made unless the conditions of the limiting condition for operation are met without reliance on provisions contained in the action requirements. As a result, Byron Unit 2 cannot startup until the Rock River level exceeds 670.6 feet MSL.

Edison's service territory is expected to see temperatures greater than 90°F through at least Monday, July 18, 1988, with today's peak load estimated at 15,500 MW. Other Commonwealth Edison units down in addition to Byron 2 include Zion 1, Crawford 8 and Collins 5. There are also deratings on Dresden, Quad Cities and LaSalle due to river and lake temperature limitations. As a result, Byron Unit 2 is needed to provide power to the service territory and needs to be returned to service as soon as possible. Current plans are to return Unit 2 to service at 3:00P.M. on July 14, 1988.

Commonwealth Edison believes that it has taken actions necessary to try to prevent an emergency situation from occurring. The proposed Technical Specification amendment was submitted as soon as the potential for such a situation was recognized, and today's unit trip could not have been anticipated. Processing the Technical Specification change under the emergency provision of 10CFR 50.91(5) will allow startup to occur with the required river flow being verified. This ensures an acceptable level of safety for unlimited continued operation while eliminating an unnecessary restriction on unit startups.

Please direct any questions you may have concerning this matter to this office.

Very truly yours,

F. G. Lentine

F. G. Lentine
PWR Licensing Supervisor

k1j

cc: Byron Resident Inspector
L. N. Olshan - NRR
Regional Administrator - RIII
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