# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE Jul, 8, 1988
COMPLETED BY W. J. Blessie
TELEPHONE 402-536-4595

MONTH June 1988

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|---|---|---|---|
| 1 | 470.7 | 17 | 465.8 |
| 2 | 470.0 | 18 | 465.0 |
| 3 | 468.1 | 19 | 458.1 |
| 4 | 466.4 | 20 | 450.5 |
| 5 | 465.8 | 21 | 436.3 |
| 6 | 465.3 | 22 | 419.4 |
| 7 | 465.7 | 23 | 411.8 |
| 8 | 465.5 | 24 | 412.7 |
| 9 | 466.4 | 25 | 413.9 |
| 10 | 467.0 | 26 | 416.0 |
| 11 | 469.4 | 27 | 415.7 |
| 12 | 470.7 | 28 | 416.9 |
| 13 | 469.0 | 29 | 416.6 |
| 14 | 466.6 | 30 | 418.9 |
| 15 | 464.1 | 31 | |
| 16 | 463.6 | | |

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.



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# OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE July 8, 1988
COMPLETED BY W. J. Blessie
TELEPHONE 402-536-4595

## OPERATING STATUS

1. Unit Name: Fort Calhoun Station — Notes
2. Reporting Period: June 1988
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   N/A
9. Power Level to Which Restricted, If Any (Net MWe): N/A
10. Reasons for Restrictions, If Any:

| | | This Month | Yr-to-Date | Cumulative |
|---|---|---|---|---|
| 11. | Hours in Reporting Period | 720.0 | 4367.0 | 129,433.0 |
| 12. | Number of Hours Reactor Was Critical | 720.0 | 4367.0 | 101,206.8 |
| 13. | Reactor Reserve Shutdown Hours | 0.0 | 0.0 | 1,309.5 |
| 14. | Hours Generator On-Line | 720.0 | 4367.0 | 100,285.8 |
| 15. | Unit Reserve Shutdown Hours | 0.0 | 0.0 | 0.0 |
| 16. | Gross Thermal Energy Generated (MWH) | 1,041,667.8 | 5,900,155.9 | 130,447,781.1 |
| 17. | Gross Electrical Energy Generated (MWH) | 340,322.0 | 1,986,104.0 | 42,959,025.2 |
| 18. | Net Electrical Energy Generated (MWH) | 323,802.8 | 1,888,598.7 | 41,032,681.5 |
| 19. | Unit Service Factor | 100.0 | 100.0 | 77.5 |
| 20. | Unit Availability Factor | 100.0 | 100.0 | 77.5 |
| 21 | Factor (Using MDC Net) | 94.1 | 90.5 | 68.8 |
| 22. | Unit Capacity Factor (Using DER Net) | 94.1 | 90.5 | 66.9 |
| 23. | Unit Forced Outage Rate | 0.0 | 0.0 | 3.0 |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    The 1988 Refueling Shutdown is tentatively scheduled for September 2, 1988 with startup tentatively scheduled for November 18, 1988.
25. If Shut Down at End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

| | | Forcast | Achieved |
|---|---|---|---|
| INITIAL CRITICALITY | | | |
| INITIAL ELECTRICITY | N/A | | |
| COMMERCIAL OPERATION | | | |

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE July 11, 1988
COMPLETED BY W. J. Blessie
TELEPHONE (402) 536-4595

# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June 1988

| No. | Date | Type[1] | Duration (Hours) | Reason[2] | Method of Shutting Down Reactor[3] | Licensee Event Report# | System Code[4] | Component Code[5] | Cause & Corrective Action to Prevent Recurrence |
|---|---|---|---|---|---|---|---|---|---|
| 88-02 | 880618 | S | Ø | H | 4 | N/A | SD | COND | On June 18, 1988, power was reduced to 90% because of a poor performing condenser coupled with high river water temperature caused high condenser back pressure. The increase in condenser back pressure, elevated the condenser hot well temperature and caused a reduction in hydrogen cooling capabilities. This reduction in cooling capabilities is limiting the electrical output of the generator. Unit remained at 90% for the duration of the report period. |

1
F-Forced
S-Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

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Exhibit 1 - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending June 1988.

1. Scheduled date for next refueling shutdown. — September 1988

2. Scheduled date for restart following refueling. — November 1988

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      Incorporate cycle specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. —

   c. If no such review has taken place, when is it scheduled? —

4. Scheduled date(s) for submitting proposed licensing action and support information. — August 1988

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:

| | | |
|---|---|---|
| a) in the core | 133 | assemblies |
| b) in the spent fuel pool | 393 | " |
| c) spent fuel pool storage capacity | 729 | " |
| d) planned spent fuel pool storage capacity | May be increased via fuel pin consolidation | " |

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1994*

*Full core offload of 133 assemblies lost.

Prepared by Kris Hatt — Date June 29, 1988

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

June 1988
Monthly Operations Report

## I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% power until high river temperature, condenser efficiency, and generator cooling problems forced reduction in power to 90% beginning June 18, 1988. Forty-four new fuel bundles were received and inspected for use during the next refueling outage. NRC inspections were performed in the areas of confirmatory measures, corrective actions program, and emergency preparedness. The annual emergency drill was held on June 8, and the annual emergency exercise was held on June 22. Construction continues on the maintenance shop, warehouse, and training facility. No safety valves or PORV challenges or failures occurred.

### A. PERFORMANCE CHARACTERISTICS

None

### B. CHANGES IN OPERATING METHODS

None

### C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

### D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

| Procedure | Description |
|---|---|
| SP-STROKE-1 | In Service Testing of Air Operated, CQE Valves.<br><br>This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it only allowed stroke testing to be conducted on 39 air operated CQE valves. The objective of the testing is to determine if valve operability was degraded (or is degrading) due to the intrusion of water into the instrument air system. This testing did not in any way compromise plant safety, but enhanced it by ensuring operability of safety related valves. |

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

| Procedure | Description |
|---|---|
| SP-SFP-FILTER-1 | Spent Fuel Pool Filter Element Removal from Spent Fuel Pool.<br><br>This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it provided a safe and orderly method for the one-time job of a filter changeout on the underwater filter unit in the spent fuel pool. A safety analysis was performed and results showed the job would pose no unreviewed safety threat. An ALARA package was written and a prejob briefing was completed with support personnel attending. The job was completed with no unexpected incidents. Although manrem for the job has not been totalled, an average dose was approximately 25 mr, much less than expected. |

System Acceptance Committee Packages for June 1988:

| Package | Description/Analysis |
|---|---|
| EEAR FC-85-179 | Replacement of HCV-247 and HCV-248.<br><br>This modification provided for the replacement of existing Valcor solenoid valves with non-Target Rock solenoid valves. This modification does not have an adverse effect on the safety analysis. |
| EEAR FC-86-023 | Discharge Tunnel Temperature Probe.<br><br>This modification provided for the reinstallation of a temperature probe in the discharge tunnel. The new installation will allow for easier removal for repairs and will prevent freezeout during cold temperatures. This temperature probe provides monitoring of discharge water temperature from the plant. This modification does not have an adverse effect on the safety analysis. |
| EEAR FC-87-064 | Phase III Security Barriers.<br><br>This modification is considered safeguards information and does not have an adverse effect on the safety analysis. |

E. RESULTS OF LEAK RATE TESTS

A leak rate test on the Personnel Air Lock (PAL) was completed on June 3, 1988, per ST-CONT-3, F.2. The "as found" and "as left" leak rates for the PAL were both 0 sccm.

The "as left" "B" and "C" leak rate was 374.2 sccm following the April 19, 1988, leak rate test. Since the PAL tested zero leakage during the June test, the total leakage remains unchanged. This leak rate is well below the allowed leakage of 0.6 $L_a$ (62,951 sccm) as specified in 10CFR50 Appendix J.

F. CHANGES IN PLANT OPERATING STAFF

Mr. Ron Arnold began his employment with OPPD in the Helper classification of the Operations department.

G. TRAINING

During June, Conduct of Maintenance training and training for Central Maintenance personnel for the upcoming refueling outage were initiated. Training for five Senior Reactor Operator candidates was completed. An Auxiliary Operator-Nuclear training program was initiated. The simulator portion of the NRC pilot requalification examination was completed for eight licensed operators.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

| Amendment No. | Description |
|---|---|
| 114 | The amendment updates the reactor coolant system pressure-temperature limits for heatup and cooldown to reflect the new fluence prediction developed and the more limiting chemistry factor associated with the 3-410 weld. |

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates

W. Gary Gates
Manager-Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102-2247
402/536-4000

July 14, 1988
LIC-88-589

U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: June Monthly Operating Report

Pursuant to Technical Specification Section 5.9.1, and 10 CFR Part 50.4(b)(1), please find enclosed one copy of the June 1988 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

K. J. Morris
Division Manager
Nuclear Operations

KJM/wjb

Enclosures

c: P. H. Harrell - NRC Senior Resident Inspector
NRC Regional Office
Office of Management & Program Analysis (2)
R. M. Caruso - Combustion Engineering
R. J. Simon - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
American Nuclear Insurers
NRC File (FCS)

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