



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 19
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Gulf States Utilities Company (the licensee) dated June 18, 1987, as supplemented March 11, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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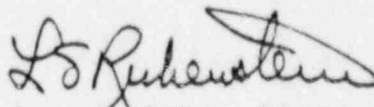
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 19 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 7, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 19

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. Overleaf page provided to maintain document completeness.

REMOVE PAGES

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INSERT PAGES

3/4 3-20
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TABLE 3.3.2-2
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level - Low Low, Level 2	≥ -43 inches*	≥ -47 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Containment Purge Isolation Radiation - High	≤ 1.3 R/hr	≤ 1.57 R/hr
2. MAIN STEAM LINE ISOLATION		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -143 inches*	≥ -147 inches
b. Main Steam Line Radiation - High	≤ 3.0 x full power background	≤ 3.6 x full power background
c. Main Steam Line Pressure - Low	≥ 849 psig	≥ 837 psig
d. Main Steam Line Flow - High		
1. Line A	≤ 146 psid	≤ 151 psid
2. Line B	≤ 156 psid	≤ 161 psid
3. Line C	≤ 153 psid	≤ 158 psid
4. Line D	≤ 164 psid	≤ 169 psid
e. Condenser Vacuum - Low	≥ 8.5 inches Hg. vacuum	≥ 7.6 inches Hg. vacuum
f. Main Steam Line Tunnel Temperature - High	$\leq 135^\circ\text{F}$	$\leq 142.5^\circ\text{F}$
g. Main Steam Line Tunnel Δ Temperature - High	$\leq 51^\circ\text{F}$	$\leq 55^\circ\text{F}$

RIVER BEND - UNIT 1

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AMENDMENT NO. 11

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>2. MAIN STEAM LINE ISOLATION (Cont'd)</u>		
h. Main Steam Line Area Temperature - High (Turbine Building)		
1. Main Steam Tunnel Area (El. 95')	$\leq 142^{\circ}\text{F}$	$\leq 145.3^{\circ}\text{F}$
2. Main Steam Tunnel Area (El. 114')	$\leq 142^{\circ}\text{F}$	$\leq 145.3^{\circ}\text{F}$
3. Main Steam Line Turbine Shield Wall	$\leq 102^{\circ}\text{F}$	$\leq 106^{\circ}\text{F}$
4. MSL Moisture Separator and Reheater Area	$\leq 126^{\circ}\text{F}$	$\leq 130^{\circ}\text{F}$
<u>3. SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low Low Level 2	$\geq - 43$ inches*	$\geq - 47$ inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Fuel Building Ventilation Exhaust Radiation - High		
IRMS*RE5A	$\leq 1.82 \times 10^3 \mu\text{Ci/sec}$	$\leq 2.18 \times 10^3 \mu\text{Ci/sec}$
IRMS*RE5B	$\leq 5.88 \times 10^{-4} \mu\text{Ci/cc}$	$\leq 7.05 \times 10^{-4} \mu\text{Ci/cc}$
d. Reactor Building Annulus Ventilation Exhaust Radiation - High	$\leq 4.32 \times 10^{-5} \mu\text{Ci/cc}$	$\leq 5.19 \times 10^{-5} \mu\text{Ci/cc}$
<u>4. REACTOR WATER CLEANUP SYSTEM ISOLATION</u>		
a. Δ Flow - High	≤ 55 gpm	≤ 62.1 gpm
b. Δ Flow Timer	≤ 45 seconds	≤ 47 seconds
c. Equipment Area Temperature - High		
1. Heat Exchanger Room	$\leq 104.5^{\circ}\text{F}$	$\leq 107.5^{\circ}\text{F}$
2. Pump Rooms A & B	$\leq 165^{\circ}\text{F}$	$\leq 169.5^{\circ}\text{F}$
3. Valve Nest Room	$\leq 110^{\circ}\text{F}$	$\leq 114.5^{\circ}\text{F}$
4. Demineralizer Rooms 1 and 2	$\leq 110^{\circ}\text{F}$	$\leq 114.5^{\circ}\text{F}$
5. Receiving Tank Room	$\leq 110^{\circ}\text{F}$	$\leq 114.5^{\circ}\text{F}$

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
4. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION (Cont'd)</u>		
d. Equipment Area Δ Temperature - High		
1. Heat Exchanger Room	$\leq 39^{\circ}\text{F}$	$\leq 42.5^{\circ}\text{F}$
2. Pump Rooms A and B	$\leq 78^{\circ}\text{F}$	$\leq 82^{\circ}\text{F}$
3. Valve Nest Room	$\leq 46^{\circ}\text{F}$	$\leq 49.5^{\circ}\text{F}$
4. Demineralizer Rooms 1 and 2	$\leq 46^{\circ}\text{F}$	$\leq 49.5^{\circ}\text{F}$
5. Receiving Tank Room	$\leq 46^{\circ}\text{F}$	$\leq 49.5^{\circ}\text{F}$
e. Reactor Vessel Water Level - Low Low Level 2	$\geq - 43$ inches*	$\geq - 47$ inches
f. Main Steam Line Tunnel Ambient Temperature - High	$\leq 135^{\circ}\text{F}$	$\leq 142.5^{\circ}\text{F}$
g. Main Steam Line Tunnel Δ Temperature - High	$\leq 51^{\circ}\text{F}$	$\leq 55^{\circ}\text{F}$
h. SLCS Initiation	NA	NA
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>		
a. RCIC Steam Line Flow - High	$\leq 127'' \text{H}_2\text{O}^{**}$	$\leq 135.5'' \text{H}_2\text{O}^{**}$
b. RCIC Steam Line Flow - High Timer	≥ 3 seconds	≤ 13 seconds
c. RCIC Steam Supply Pressure - Low	≥ 60 psig	≥ 55 psig
d. RCIC Turbine Exhaust Diaphragm Pressure - High	≤ 10 psig	≤ 20 psig
e. RCIC Equipment Room Ambient Temperature - High	$\leq 182^{\circ}\text{F}$	$\leq 186.4^{\circ}\text{F}$
f. RCIC Equipment Room Δ Temperature - High	$\leq 96^{\circ}\text{F}$	$\leq 99^{\circ}\text{F}$

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u> (Cont'd)		
g. Main Steam Line Tunnel Ambient Temperature - High	$\leq 135^{\circ}\text{F}$	$\leq 142.5^{\circ}\text{F}$
h. Main Steam Line Tunnel Δ Temperature - High	$\leq 51^{\circ}\text{F}$	$\leq 55^{\circ}\text{F}$
i. Main Steam Line Tunnel Temperature Timer	0 seconds	NA
j. RHR Equipment Room Ambient Temperature - High	$\leq 117^{\circ}\text{F}$	$\leq 121.1^{\circ}\text{F}$
k. RHR Equipment Room Δ Temperature - High	$\leq 29^{\circ}\text{F}$	$\leq 33.6^{\circ}\text{F}$
l. RHR/RCIC Steam Line Flow - High	$\leq 60.7'' \text{H}_2\text{O}$	$\leq 64.2'' \text{H}_2\text{O}$
m. Drywell Pressure - High	$\leq 1.68 \text{ psig}$	$\leq 1.88 \text{ psig}$
n. Manual Initiation	NA	NA
6. <u>RHR SYSTEM ISOLATION</u>		
a. RHR Equipment Area Ambient Temperature - High	$\leq 117^{\circ}\text{F}$	$\leq 121.1^{\circ}\text{F}$
b. RHR Equipment Area Δ Temperature - High	$\leq 29^{\circ}\text{F}$	$\leq 33.6^{\circ}\text{F}$
c. Reactor Vessel Water Level - Low Level 3	$\geq 9.7 \text{ inches}^*$	$\geq 8.7 \text{ inches}$
d. Reactor Vessel Water Level - Low Low Low Level 1	$\geq - 143 \text{ inches}^*$	$\geq - 147 \text{ inches}$