

ALABAMA POWER COMPANY  
JOSEPH M. FARLEY NUCLEAR PLANT  
UNIT 1 - ANNUAL REPORT  
REQUIRED BY 10CFR50.59

Section 50.59 of Part 50, Licensing of Production and Utilization Facilities, of the regulations of the United States Nuclear Regulatory Commission, states that the holder of a license authorizing operation of a production or utilization facility may (1) make changes in the facility as described in the safety analysis report, and (2) make changes in the procedures as described in the safety analysis report, and (3) conduct tests or experiments not described in the safety analysis report, without prior commission approval, unless the proposed change, test or experiment involves a change in the technical specifications incorporated in the license or an unreviewed safety question (as defined in 10CFR50.59).

The licensee is required to maintain records of such changes, tests or experiments, and those records are required to include written safety evaluations which provide the basis for the determination that the changes, tests or experiments do not involve any unreviewed safety questions.

Brief descriptions and a summary of the safety evaluations of the changes, tests or experiments as described above, for the Joseph M. Farley Nuclear Plant Unit 1 which were completed in 1987, are provided in the following.

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Subject: PCR/PCN 84-2542 (S81-1-2542)

Description: Routed the steam generator blowdown (SGBD) and steam generator outlet (SGO) samples from the steam sample panel to the rinse water sink in the high activity laboratory. The rinse water sink drains to the floor drain tank which is used for low level radioactive liquid.

Safety Evaluation: The effluent from the rinse water sink is monitored for any activity prior to being transferred to the waste monitor tank. During normal plant operation, the SGBD and SGO samples would have very low radioactivity levels. In the unlikely event of a steam generator tube leak and contamination of the SGBD and SGO samples, the effluent from the rinse water sink will be transferred to the waste monitor tank and processed through the waste evaporator or disposable demineralizer as required.

PORC Review: PORC Meeting 1537, 6/24/86  
PORC Meeting 1555, 8/12/86

Subject: PCR/PCN 84-2609 (S84-0-2609)

Description: Upgraded the existing Primary Meteorological Tower Instrumentation. This included the addition of redundant wind speed and wind direction channels at the 150 foot elevation.

Safety Evaluation: This modification increases the reliability of the Primary Meteorological Tower Instrumentation. Also, the addition of redundant wind speed and wind direction instruments prevents entering a Technical Specification Limiting Condition for Operation upon failure of a single channel.

PORC Review: PORC Meeting 1522, 5/13/86

Subject: PCR/PCN 84-2960 (S84-1-2960)

Description: Replaced the existing carbon steel piping in the moisture separator reheater (MSR) first stage extraction steam line with stainless steel.

Safety Evaluation: This modification affects only the MSR first stage extraction steam piping which is not a safety related component. The stainless steel piping is more resistant to erosion and corrosion, thus it will enhance the design life and operation of the extraction steam system.

PORC Review: PORC Meeting 1511, 4/17/86

Subject: PCR/PCN 84-2961 (S84-1-2961)

Description: Replaced the existing carbon steel extraction steam piping to the 6A and 6B feedwater heaters with stainless steel piping.

Safety Evaluation: This modification affects only the extraction steam piping to feedwater heaters 6A and 6B which is not a safety related component. The stainless steel piping is more resistant to erosion and corrosion thus it will enhance the design life and operation of the extraction steam system.

PORC Review: PORC Meeting 1513, 4/22/86

Subject: PCR/PCN 85-3084 (S85-1-3084)

Description: Modified the existing nuclear steam supply system (NSSS) sampling station arrangement to provide accessibility to the sample room following an accident. This modification involved the addition of a fume hood and sample station for the reactor samples and increased the ventilation for the post accident sampling system (PASS) hood. Sample streams for reactor coolant, pressurizer steam and liquid, volume control tank gas, and the accumulators will be routed from the reactor sample station to the fume hood. A new sample control panel is installed in the high activity laboratory.

Safety Evaluation: This modification will ensure accessibility to the sample room following an accident.

PORC Review: PORC Meeting 1381, 4/18/85  
PORC Meeting 1491, 2/20/86

Subject: PCR/PCN 85-3124 (B85-1-3124)

Description: Provides for the addition of a new vent valve on the waste evaporator condensate tank.

Safety Evaluation: This valve will simplify the process of educting air and other non-condensibles from beneath the diaphragm of the tank. The addition of this valve will not degrade the safe operation of this system.

PORC Review: PORC Meeting 1624, 1/15/87

Subject: PCR/PCN 85-3267 (S85-1-3267)

Description: Installed a new sewage treatment plant effluent line to the waste settling pond and capped the existing effluent line to the service water discharge.

Safety Evaluation: This modification will allow the effluent to discharge into the waste settling pond instead of the service water system. This will prevent backflow from the service water dilution line. The sewage treatment facility does not affect nuclear safety.

PORC Review: PORC Meeting 1595, 11/11/86

Subject: PCR/PCN 85-3357 (B85-1-3357)

Description: Modified the outlet piping of radiation monitor RE-16 to allow the use of a decontamination tool. Also, the outlet piping was modified to reduce the contamination build up in the radiation monitor.

Safety Evaluation: This modification would reduce the time required to decontaminate the monitor while improving the reliability and sensitivity of the monitor.

PORC Review: PORC Meeting 1635, 2/5/87

Subject: PCR/PCN 85-3271 (S85-1-3271)

Description: Assigned new TPNS numbers to the feedwater flow local indicators due to duplication with indicators on the main control board.

Safety Evaluation: Drawing change only.

PORC Review: PORC Meeting 1491, 2/20/86

Subject: PCR/PCN 86-3496 (S86-0-3496)  
Description: Erected a solidification and dewatering facility (SDF).

Safety Evaluation: The SDF provides the capability to transfer primary resin, steam generator blowdown resin, chemical drains and evaporator bottoms to four bulk disposal liners for solidification or dewatering. No new liquid or gaseous effluent pathways were created. The four disposal liners are located in shielded pits which are vented to the Unit 1 Auxiliary Building Radwaste Area Ventilation System. The building sump is pumped to the Unit 1 Auxiliary Building Floor Drain System.

The SDF also provides for interim storage of up to 1690 gallons of contaminated oil.

The operation of this facility does not affect nuclear safety.

PORC Review: PORC Meeting 1554, 8/8/86  
PORC Meeting 1627, 1/23/87  
PORC Meeting 1646, 3/10/87  
PORC Meeting 1663, 4/20/87

Subject: PCR/PCN 86-3523 (B86-0-3523)

Description: Removed all the existing equipment in the decontamination room and installed a high pressure water spray decontamination unit. Items included in this unit are a stainless steel sink, several stainless steel shelves, and two receptacles for the spray unit. Also, a vent valve was added on the demineralized water system supply piping to the high pressure water spray decontamination system.

Safety Evaluation: This new system replaced an out-moded decontamination system. The addition of a vent valve will provide the ability to vent the piping during a hydrostatic test or during initial operation of the decontamination system. This new water spray unit will enhance equipment decontamination and decrease the dependency on respiratory protection by decontamination workers.

PORC Review: PORC Meeting 1611, 12/4/86  
PORC Meeting 1624, 1/15/87  
PORC Meeting 1645, 3/5/87

Subject: PCR/PCN 86-3635 (S86-0-3635)

Description: Replaced an auxiliary relay in the control circuit of well water pump #3 with a time delay to prevent tripping the pump from momentary surges in header pressure.

Safety Evaluation: This modification did not involve any safety related equipment. This will improve the reliability of well water pump #3.

PORC Review: PORC Meeting 1606, 11/25/86

Subject: PCR/PCN 86-3763 (B86-1-3763)

Description: Changed the cooling water supplies to the gear oil and bearing oil coolers of the charging/high head safety injection pumps from service water to component cooling water. Stainless steel piping was used throughout this modification except for the cross tie headers and the supply and return lines to and from the skids.

Safety Evaluation: The conversion from service water to component cooling water for cooling of the charging pump gear and bearing oil coolers should prevent future problems from overheating due to clogging of the coolers.

PORC Review: PORC Meeting 1572, 9/30/86  
PORC Meeting 1576, 10/7/86  
PORC Meeting 1580, 10/14/86  
PORC Meeting 1597, 11/15/86  
PORC Meeting 1606, 11/25/86

Subject: PCR/PCN 86-3929 (S86-1-3929)

Description: Installed carpet in the Shift Foreman's office and the Unit 1 portion of the main control room at-the-controls area. The elevated portions which constitute the operator and shift supervisor work stations are carpeted as well.

Safety Evaluation: The carpet will reduce operator fatigue and reduces the noise level in the control room. The addition of the carpet will not significantly affect the fire safety analysis of the control room. This design change will not interfere with or degrade other systems.

PORC Review: PORC Meeting 1667, 4/28/87

Subject: PCR/PCN 87-4026 (S87-1-4026)

Description: Revised drawings D-175042 sheet 6 and U-169531 to correct drawing discrepancies in the oxygen supply to the hydrogen recombiners.

Safety Evaluation: Drawing change only.

PORC Review: PORC Meeting 1656, 4/7/87

Subject: PCR/PCN 87-4351 (B87-1-4351)

Description: Installed Teledyne big-beam 8-hour emergency lights to provide light for manual operation of MOV3094A-B and MOV3094B-A (spent fuel pool cooling to the opposite train of component cooling water) located in rooms 445 and 422. Also, lights were installed to facilitate operator access to these rooms.

Safety Evaluation: The installation of these 8-hour emergency lights meets the requirement of 10CFR50, Appendix R.

PORC Review: PORC Meeting 1699, 7/10/87

Subject:

PCR/PCN 87-4384 (B87-0-4384)

Description:

Replaced the existing commercial grade Agastat relay Model 7012PA with a seismically qualified Agastat relay Model E7012PA002. The new relay is installed in the Unit 2 600V emergency load center 2E and is part of the overcurrent protection of the preferred power supply to Unit 2 - Train B 600V emergency load center 2E. It performs a safety related function. However, for Unit 1, this relay is part of the overcurrent protection of the alternate power supply to the normal section of 600V load center 1A, and it does not perform a safety related function.

Safety Evaluation:

The new relay E7012PA002 is seismically qualified by testing and is documented in test reports. These test reports show that the new relay is qualified to higher accelerations than required for all modes of relay operation. In addition, the new relay installed on Unit 2 does not affect the load center's original seismic qualification since the location, mass, and mounting do not change from a structural standpoint.

PORC Review:

PORC Meeting 1711, 8/4/87



Subject: FNP-0-AP-19

Description: Deleted this administrative procedure which consisted of a list of procedures that implemented various chapters of the Operations Quality Assurance Policy Manual (OQAPM).

Safety Evaluation: FNP-0-AP-19 was developed as an administrative tool. Its purpose was to aid in establishing the commitments stated in the OQAPM in administrative procedures or startup standards. Currently, changes in the OQAPM are implemented through administrative procedures by the manager responsible for the affected procedure.

PORC Review: PORC Meeting 1621, 1/9/87

Subject: FNP-0-AP-76 Revision 4

Description: Authorized the use of morpholine/boric acid in the secondary water chemistry control system.

Safety Evaluation: Morpholine will result in better pH control in the steam and condensate system, which in turn will reduce the amount of corrosion products transported to the steam generators. The joint use of hydrazine, boric acid, and morpholine does not create corrosive conditions for steam generator or feedtrain materials.

PORC Review: PORC Meeting 1760, 11/3/87

Subject: FNP-0-CCP-708 TCN 2D, TCN 3A, and TCN 3D

Description: TCNs 2D and 3A provided for the continuous chlorination of the Unit 1 service water system using sodium hypochlorite to reduce the number of Corbicula (clams) in the system. No biocide will be added to the Unit 2 service water system during this treatment period of Unit 1. TCN 3D allowed the addition of chlorine dioxide (biocide) to Unit 2 to be continued during the hypochlorination of Unit 1.

Safety Evaluation: Sodium hypochlorite addition can be quickly terminated if deemed necessary. Using sodium hypochlorite eliminates the personnel safety hazards associated with chlorine gas. Emergency actions and precautions already exist in this procedure. The use of both chlorine dioxide and sodium hypochlorite for prevention of fouling in the service water system is already addressed in the FSAR.

PORC Review: PORC Meeting 1678, 5/28/87  
PORC Meeting 1726, 9/4/87  
PORC Meeting 1762, 11/5/87

Subject: FNP-1-ETP-3009 Revision 0

Description: Modified the intended function of the Access Control Area air handling unit to provide adequate cooling to the Dosimetry Laboratory.

Safety Evaluation: The FSAR (paragraph 9.4.2.3.3) states that the Access Control Area HVAC system is not an Engineered Safeguard System and no credit is taken for its operation in analyzing the consequences of any accident. The results of an engineering evaluation showed that flow from the non-radwaste area to the radwaste area would be maintained as desired.

PORC Review: PORC Meeting 1702, 7/17/87

Subject: FNP-1-SOP-16.1 Revision 11, TCN 11A

Description: Defeated the low service water dilution flow trip of 1-BD-RCV-023B during Mode 5 or Mode 6 operation. Normally, this valve will trip closed on low service water dilution flow rate thus isolating the steam generator blowdown (SGBD) path to the environment.

Safety Evaluation: Service Water dilution flow is continuously recorded for each unit in the Main Control Room. SGBD releases will be calculated and authorized on a batch release permit to ensure that the total dilution flow from both units is sufficient to ensure that Technical Specification dose limits are not exceeded. SGBD will be caution tagged to require that blowdown be secured if the minimum dilution of the release permit is not met. The total dilution flow will be logged every 15 minutes. Defeating the low service water flow trip does not affect the ability of 1-BD-RCV-023B to close if a high activity level is detected in the SGBD flowpath.

PORC Review: PORC Meeting 1760, 11/3/87

Subject: FNP-0-SOP-34.0 Revision 9, TCN 9A

Description: Isolated the normal hydrogen supply to the Auxiliary Building and installed a temporary supply from a compressed hydrogen bottle. The hydrogen is supplied through a special adaptor installed in place of the bonnet of valve N1G22V029A. This can be done in order to search for leaks in the hydrogen supply piping.

Safety Evaluation: The normal volume control tank (VCT) purge will be stopped to limit the amount of makeup hydrogen required. The temporary hydrogen supply will provide sufficient pressure in the VCT for reactor coolant pump seal operation as well as hydrogen makeup to replace leakage and hydrogen lost to oxygen scavenging. Since the normal supply has a much greater capacity, a leak in the temporary supply will not create a hazard more severe than a leak in the normal supply.

PORC Review: PORC Meeting 1721, 8/27/87

Subject: FNP-1-SOP-50.0 Revision 22, TCN 22A

Description: Deleted the requirement of running a reactor coolant drain tank (RCDT) pump continuously.

Safety Evaluation: This change will reduce wear on the RCDT pumps and provide for greater reliability. System parameters are provided at the control station for the RCDT system and are normally monitored and logged approximately every four hours. Also, a high pressure alarm and a high tank level alarm are provided. When required, one pump can be started to maintain parameters in the required ranges.

PORC Review: PORC Meeting 1668, 4/30/87

Subject: FNP-1-SOP-50.4 Revision 12

Description: Deleted the requirement to backflush or fluff a demineralizer after new resin has been added.

Safety Evaluation: The existing requirement uses water in the Spent Resin Storage Tank to fluff the new resin. Deleting this requirement will increase both the effectiveness and the life of the resin by not exposing it to contamination before it is used to process fluid.

PORC Review: PORC Meeting 1698, 7/9/87

Subject: MD 86-1648

Description: Changed the setpoint for the high letdown flow alarm. The previous alarm setpoint was 120 gpm. The revised setpoint is 135 gpm.

Safety Evaluation: The letdown flow rate is currently 128 gpm with two letdown orifices in service. This is greater than the previous alarm setpoint. Changing this alarm setpoint will not affect operation of the unit under normal or accident conditions.

PORC Review: PORC Meeting 1620, 1/7/87

Subject: MD 87-1734

Description: Provided a temporary source of hydrogen to the volume control tank (VCT). The normal supply of hydrogen was isolated for repairs.

Safety Evaluation: The temporary supply will provide sufficient hydrogen pressure. The temporary supply is located in a room through which the normal supply line passes. The normal supply has a greater capacity than the temporary supply. Therefore, a leak in the temporary supply will not create a hazard more severe than a leak in the normal supply. The temporary supply will be monitored periodically.

PORC Review: PORC Meeting 1721, 8/27/87  
PORC Meeting 1722, 8/28/87

Subject: MD 87-1770

Description: Installed jumpers to allow the operation of the 1A radioactive waste area exhaust fan with the supply fan out of service. The jumpers were removed when the supply fan was returned to service.

Safety Evaluation: During this temporary minor departure from plant design, the boric acid tank area temperature was monitored to ensure the temperature remained greater than 65°F while the exhaust fan was being run. Also, a caution tag was placed on the exhaust fan control switches requiring them to be shutdown when the outside air temperature was less than 20°F. Appropriate doors between the radiation control area (RCA) and the outside air were opened to allow ventilation.

PORC Review: PORC Meeting 1758, 11/2/87  
PORC Meeting 1767, 11/12/87

Subject: FNP Emergency Plan

Description: TCN 12A changed the requirement for the Emergency Director to possess a Senior Reactor Operator's license. TCN 12B alleviated the responsibility of the Public Information Site Coordinator (PISC) to activate the Corporate Communications Department Emergency Organization. Instead, this responsibility was delegated to the Public Information Emergency Coordinator (PIEC) and the Public Information Activation Assistant. TCN 12C replaced the NOAA Weather Alert Radio System for Alert and Notification with a new tone alert system.

Safety Evaluation: With regard to TCN 12A, persons who may serve as Emergency Director must meet the following requirements: (1) Possess the training and experience required for their position as delineated in ANSI 18.1-1971; (2) occupy a position which has been designated by the General Manager - Nuclear Plant; and (3) have completed Emergency Director Training. The Emergency Director Training meets the requirements of NUREG-0654 and 10CFR50 Appendix E. With regard to TCN 12B, the existing plan may cause the PISC to be unduly delayed in his attempt to travel to the site; therefore, the changes will ensure that the PISC will be available to travel to the site in a timely manner should the Emergency Operations Facility be activated. With regard to TCN 12C, the new system will add confidence that the public inside the ten mile Emergency Planning Zone (EPZ) and outside any siren-covered zones will keep their radio in service, thus ensuring a more reliable alert and notification system. (Note: a

significant percentage of the public who were issued NOAA weather alert radios were taking their radios out of service due to excessive activations from NOAA to notify of severe weather conditions).

PORC Review: PORC Meeting 1670, 5/5/87 (TCN 12A)  
PORC Meeting 1723, 9/1/87 (TCN 12B)  
PORC Meeting 1724, 9/2/87 (TCN 12C)

Subject: FP-ALA-FE-6 Rev. 0

Description: Provided procedural guidance for inspection, measurements, and removal of grid tabs from fuel assemblies in the spent fuel pool.

Safety Evaluation: The equipment which comes in contact with the fuel is controlled and does not place excessive force on the fuel assemblies. Fuel is moved using only the spent fuel pool bridge crane and the fuel handling tool. None of the equipment moved over the fuel weighs more than 3000 pounds.

PORC Review: PORC Meeting 1632, 2/3/87

Subject: FP-ALA-FE-6 TCN 0B

Description: Provided instructions for removal and collecting of grid vanes from an irradiated Optimized Fuel Demonstration Assembly (OFDA).

Safety Evaluation: The equipment which comes in contact with the fuel is controlled and does not place excessive force on the assembly. The fuel will be moved only with the spent fuel pool bridge crane and the fuel handling tool. None of the equipment moved over the fuel weighs more than 3000 pounds.

PORC Review: PORC Meeting 1637, 2/2/87

Subject: Safety Evaluation: FSAR Stroke Time Review

Description: Revised the closure times shown in the FSAR for several containment isolation valves to be consistent with the closure times shown in the Technical Specifications.

Safety Evaluation: An analysis determined that revising the closure times for these valves will not increase the post-accident radiation dose levels above the applicable limits.

PORC Review: PORC Meeting 1795, 12/31/87

NOTE: The following changes were incorporated into the FSAR as part of Revision 5, July 1987.

Subject: Safety Evaluation: Update of FSAR Section 9.4.7.4

Description: Deletes the requirement for testing those components in the Diesel Building heating and ventilation system which are normally in use.

Safety Evaluation: This change does not decrease the reliability of the heating and ventilation system at the Diesel Building.

PORC Review: PORC Meeting 1694, 6/30/87

Subject: Safety Evaluation: FSAR PC 86-15

Description: Revised some of the specifications for the water treatment system which are specified in FSAR section 9.2.8.2. These specifications had been stated incorrectly.

Safety Evaluation: This is a change to the text of the FSAR to correct errors in the description of a nonsafety related system. No modifications were made to any equipment.

PORC Review: PORC Meeting 1694, 6/30/87

Subject: Safety Evaluation: FSAR Section 8.3.1.1.4

Description: Corrected section 8.3.1.1.4 of the FSAR. The text of this section had disagreed with the information in FSAR figures 8.3-23 and 8.3-24 and the "as-built" condition of the equipment involved. The corrected information deals with the backup power sources for the 120 volt AC vital instrumentation power system.

Safety Evaluation: The FSAR figures are correct and reflect the "as-designed" and "as-built" condition of the equipment. Correcting the text is administrative in nature.

PORC Review: PORC Meeting 1694, 6/30/87



Subject: Safety Evaluation: FSAR PC 86-08

Description: Corrected a discrepancy concerning the location of reactor coolant pump (RCP) vibration displays. RCP vibration displays are located in the electrical penetration room and there is an associated annunciator in the main control room. FSAR section 5.5.1.2(c) had indicated that RCP vibration is displayed in the control room.

Safety Evaluation: This change provided a needed correction to the FSAR.

PORC Review: PORC Meeting 1694, 6/30/87

Subject: Safety Evaluation: FSAR PC 86-10

Description: Corrected a discrepancy in FSAR section 6.5.5. This section had stated that the auxiliary feedwater flow indicators were environmentally qualified. The flow transmitters associated with these indicators are environmentally qualified but the flow indicators themselves are not environmentally qualified.

Safety Evaluation: This change provided a needed correction to the FSAR. The indicators are in a mild environment and are not required to be environmentally qualified.

PORC Review: PORC Meeting 1694, 6/30/87

Subject: Safety Evaluation: FSAR Sections 8.3.1.1.2 and 8.3.1.1.6

Description: Updated FSAR sections 8.3.1.1.2 and 8.3.1.1.6 to take into account additions to the plant's electrical system.

Safety Evaluation: This provided a needed correction to the FSAR.

PORC Review: PORC Meeting 1694, 6/30/87

Subject: Safety Evaluation: Containment Free Volume

Description: Revised FSAR paragraph 3.8.1.1.a to remove a discrepancy between this paragraph, FSAR Table 6.2-1, and Technical Specification paragraph 5.2.1.g. FSAR paragraph 3.8.1.1.a had used a larger value for free volume of the containment than the other two sources.

Safety Evaluation: Using the lower value of containment free volume is conservative and consistent with the other sources.

PORC Review: PORC Meeting 1694, 6/30/87

Subject: Safety Evaluation: New Appendix 9B to FSAR Which Incorporated the FPPR

Description: Incorporated the fire protection program into the FSAR as requested by Generic Letter 86-10. The fire protection program had been in a separate document called the Fire Protection Program Reevaluation. The program is now in FSAR Appendix 9B. Appropriate additions and deletions were made to ensure that the new appendix is in the proper format and meets applicable requirements. In addition, cable insulation combustible loadings were revised as required.

Safety Evaluation: These changes do not decrease the effectiveness of the fire protection program. Changes to the combustible loading were evaluated and found to be acceptable.

PORC Review: PORC Meeting 1691, 6/25/87

Subject: GO-EIP-101, Rev. 4, Rev. 5

Description: This procedure delineates the organization to be implemented by the general office in support of an emergency.

Safety Evaluation: Revision 4 to the procedure reflects management reorganization within Nuclear Generation.

Revision 5 to the procedure indicates title and facility name changes.

Subject: GO-EIP-102, Rev. 3, Rev. 4

Description: This procedure delineates the organization to be implemented and facilities to be utilized by the corporate communication department in support of an emergency.

Safety Evaluation: Revision 3 to the procedure adds a QA records section and reflects several Corporate Communication title changes.

Revision 4 to the procedure centralizes the activities of the Corporate Communications Department into one procedure.

Subject: GO-EIP-111, TCN-11A, TCN-11B, Rev. 12,  
Rev. 14, TCN-14A, Rev. 15, Rev. 16

Description: This procedure delineates the steps  
necessary to activate the Nuclear Generation  
Department Emergency Organization and provide  
notification of the event.

Safety Evaluation: Change TCN-11A adds Environmental  
Affairs to the call list in the event of a  
transformer fire which releases PCBs.

Change TCN-11B adds a table for  
emergency coordinator notification in the event of  
a waste transportation accident not requiring EOF  
action.

Revision 12 to the procedure  
incorporates organizational changes in the offsite  
Technical Support Organization which combines the  
duties of the Manager of Nuclear Engineering and  
Technical Support and the Manager of Nuclear  
Operations and Administration into the position of  
General Manager - Nuclear Support. These combined  
offsite support responsibilities are divided among  
four managers reporting to the General Manager -  
Nuclear Support. Phone number changes were also  
made.

Revision 14 to the procedure updates the  
personnel and phone number listings.

Change TCN-14A to the procedure involves  
phone number changes only.

Revision 15 to the procedure involves  
phone number changes, title changes, personnel  
changes, procedural clarification changes, changes  
to reflect new EOC equipment, and changes to  
reference the use of the new helicopter.  
Additionally, a change is incorporated in the  
EOF Activation Checklist which will prompt the  
Emergency Coordinator to call the plant and  
request a list of available plant personnel to be  
used to supplement EOF staffing.

Revision 16 updates phone number, position titles,  
and personnel changes. Procedural  
clarifications were made and a step for the EC to  
notify Alabama Radiological Health was deleted  
because FNP-EIP-26 calls for the Emergency  
Director to take this action.

Subject: GO-EIP-112, Rev. 6, Rev. 6A, Rev. 7

Description: This procedure provides instructions for establishing communications between emergency support personnel in transit and personnel located at company facilities.

Safety Evaluation: Revision 6 to the procedure incorporates changes made to the Nuclear Generation management organization.

Revision 6A to the procedure involves phone and radio number changes.

Revision 7 to the procedure involves phone number changes, radio call number changes, and title changes.

Subject: GO-EIP-114, Rev. 6

Description: The purpose of these procedure is to provide guidance in the coordination and distribution of news releases in an emergency condition.

Safety Evaluation: Revision 6 to the procedure involves changes to agency and personnel titles, adding the Alabama Bureau of Radiological Health to the list for notification of news release content prior to release and adding the Public Information Emergency Coordinator as an approval for news releases.

Subject: GO-EIP-116, Rev. 3, Rev. 4

Description: This procedure provides guidance for initially staffing the EOF or changing shifts in the EOF during an emergency.

Safety Evaluation: Revision 3 to the procedure incorporates changes in the Nuclear Generation management organization.

Revision 4 to the procedure involves title changes and procedure reference changes to make the Dose Assessment Director turnover more effective.

Subject: GO-EIP-117, Rev. 5  
Description: This procedure delineates the Emergency Operations Facility administrative activities.  
Safety Evaluation: Revision 5 to the procedure involves phone number and title changes.

Subject: GO-EIP-118, Rev. 4, Rev. 5  
Description: This procedure delineates the steps necessary to activate the Corporate Communication Department Emergency Organization and provide notification of an event.

Safety Evaluation: Revision 4 to the procedure involves title changes, phone number changes, and procedural format changes.  
Revision 5 to the procedure incorporates an Activation Assistant, revises methodology to include the Activation Assistant, and adds necessary checklists for emergency positions.

Subject: GO-EIP-119, Rev. 3  
Description: This procedure delineates the criteria and authority for relocating the EOF from the onsite EOF to the Wiregrass District Office.  
Safety Evaluation: Revision 3 to the procedure includes changes to the equipment list and details of room layouts.

Subject: GO-EIP-121, Rev. 0  
Description: This procedure provides guidance on the activities and responsibilities for Corporate Communication emergency positions.  
Safety Evaluation: This procedure is a new procedure that will be used to provide guidance on the necessary activities and responsibilities for several Corporate Communication emergency positions.

Subject: GO-EIP-131, Rev. 7, Rev. 8

Description: This procedure establishes the actions to be taken to ensure the operational readiness of EOC equipment and supplies.

Safety Evaluation: Revision 7 to the procedure incorporates changes within the Nuclear Generation management organization.

Revision 8 to the procedure involves the changing of a file number.

Subject: GO-EIP-132, Rev. 5

Description: This procedure establishes guidance for the conduct of drills and exercises to maintain emergency preparedness.

Safety Evaluation: Revision 5 to the procedure incorporates changes within the Nuclear Generation management organization.

Subject: GO-EIP-134, Rev. 6

Description: This procedure delineates the responsibilities for conducting emergency plan training for corporate personnel.

Safety Evaluation: Revision 6 to the procedure incorporates changes within the Nuclear Generation management organization.

Subject: GO-EIP-135, Rev. 3, Rev. 4

Description: This procedure delineates responsibilities for Emergency Plan review and revision.

Safety Evaluation: Revision 3 to the procedure incorporates changes within the Nuclear Generation management organization.

Revision 4 to the procedure involves title changes only.

Subject: GO-EIP-136, Rev. 3

Description: This procedure defines the responsibilities of the Corporate Communication Department regarding weather alert radio distribution and maintenance.

Safety Evaluation: Revision 3 to the procedure incorporates changes within the Nuclear Generation management organization.

Subject: GO-NG-2, Rev. 3, Rev. 4

Description: This procedure describes the organization and responsibilities of the General Office Nuclear Support section.

Safety Evaluation: Revision 3 to the procedure is an organizational change reflecting combined duties of:

- Manager Nuclear Engineering and Technical Support; and
- Manager Nuclear Operation and Administration

into General Manager Nuclear Support.

Revision 4 changes the revision number only. The existing revision number 3 was inadvertently used instead of revision 4.

Subject: GO-NG-5, Rev. 3

Description: This procedure describes organizational functions and responsibilities for conducting an environmental monitoring program.

Safety Evaluation: Revision 3 to the procedure is an editorial change to reflect the change in title from Plant Manager to General Manager - Nuclear Plant.

Subject: GO-NG-44, Rev. 3

Description: This procedure establishes the guidelines to be used by Nuclear Support for the coordination of inservice inspection activities in support of Farley Nuclear Plant.

Safety Evaluation: Revision 3 to the procedure implements changes to show specific due dates, the appropriate NRC addressee, and the requirement source for each submittal. The duties of the Senior Project Engineer - ISI were also clarified to include the review and amendment of the FSAR and Technical Specifications requirements as required.



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Alabama Power  
*the southern electric system*

10CFR50.59

March 31, 1988

Docket No. 50-348

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555

Gentlemen:

Attached for your review is the annual report required by 10CFR50.59 for 1987. This report summarizes changes to the plant performed in accordance with the provisions of 10CFR50.59 for Joseph M. Farley Nuclear Plant Unit 1.

If you have any questions, please advise.

Respectfully submitted,

R. P. McDonald

RPM/REM:dst-D1.46

Attachments

cc: Mr. L. B. Long  
Dr. J. N. Grace  
Mr. E. A. Reeves  
Mr. W. H. Bradford

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