

ATTACHMENT 1

PROPOSED CHANGES TO APPENDIX A

TECHNICAL SPECIFICATION FOR

QUAD CITIES STATION UNIT 2

FACILITY OPERATING LICENSE DPR-30

REVISED PAGES: 3.2/4.2-15 (DPR-30)  
3.2/4.2-18 (DPR-30)  
3.2/4.2-18a (DPR-30)

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TABLE 3.2-4

POSTACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS<sup>[2]</sup>

Minimum Number of Operable Chan- nels <sup>[1]</sup> <sup>[3]</sup>	Parameter	Instrument Readout Location Unit 2	Number Provided	Range
1	Reactor pressure	902-2	1 2	0-1500 psig 0-1200 psig
1	Reactor water level	902-3	2	-340 inches+60 inches
1	Torus water temperature	902-21	2	0-200°F
1	Torus air temperature	902-21	2	0-600°F
	Torus water level indicator	902-3	1	-5 inches +5 inches (narrow range)
2 <sup>[6]</sup>	Torus water level indicator	902-3	2	0-30 feet (wide range)
	Torus water level sight glass		1	18 inch range (narrow range)
1	Torus pressure	902-3	1	-5 inches Hg to 5 psig
2	Drywell pressure	902-3	1	-5 inches Hg to 5 psig -10 inches Hg to 70 psig
			2	-5 to 250 psig
2	Drywell temperature	902-3	6	0-600°F
2	Neutron monitoring	902-5	4	0.1-10 <sup>8</sup> CPS
2 <sup>[4]</sup>	Torus to drywell differential pressure		2	0-3 psid
1 <sup>(8)</sup>	Drywell Hydrogen concentration	902-55, 56	2	0-4%
2 <sup>(7)</sup>	Drywell radiation monitor	902-55, 56	2	1 to 10 <sup>8</sup> R/hr
2/valve <sup>[5]</sup>	Main Steam RV posi- tion, acoustic monitor	902-21	1 per valve	NA
	Main Steam RV position, temperature monitor	902-21	1 per valve	0-600°F
2/valve <sup>[5]</sup>	Main Steam SV posi- tion, acoustic monitor	902-21	1 per valve	NA
	Main Steam SV position, temperature monitor	902-21	1 per valve	0-600°F

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TABLE 4.2-2

POSTACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

Minimum Number of Operable Channels*	Parameter	Instrument Readout Location Unit 2	Calibration	Instrument Check
1	Reactor pressure	902-5	Once every 3 months	Once per day
1	Reactor water level	902-3	Once every 3 months	Once per day
1	Torus water temperature	902-21	Once every 3 months	Once per day
1	Torus air temperature	902-21	Once every 3 months	Once per day
2	Torus water level (narrow range)	902-3	Once every 3 months	Once per day
	Torus water level indicator (wide range)	902-3	Once every 18 months	Once per 31 days
	Torus water level sight glass		N/A	None
1	Torus pressure	902-3	Once every 3 months	Once per day
2	Drywell pressure	902-3	Once every 3 months	Once per day
2	Drywell temperature	902-3	Once every 3 months	Once per day
2	Neutron monitoring	902-5	Once every 3 months	Once per day
2	Torus to drywell differential pressure		Once every 6 months	None
1	Drywell hydrogen concentration	902-55, 56	Once every 3 months	Once per 31 days
2	Drywell radiation monitor	902-55, 56	Once every 18 months ***	Once per 31 days
2/valve	Main Steam RV Position, acoustic monitor	902-21	**	Once per 31 days
	Main Steam RV Position, temperature monitor	902-21	Once every 18 months	Once per 31 days

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TABLE 4.2-2 (con'd)

Minimum Number of Operable Channels*	Parameter	Instrument Readout Location Unit 2	Calibration	Instrument Check
2/valve	Main Steam SV Position, acoustic monitor	902-21	**	Once per 31 days
	Main Steam SV Position, temperature monitor	902-21	Once every 18 months	Once per 31 days

\* Instrument channels required during power operation to monitor postaccident conditions.

\*\* Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular safety or relief valve.

\*\*\* Calibration shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/hr; and a one-point calibration check of the detector below 10 R/hr with an installed or portable gamma source.

## ATTACHMENT 2

### SUMMARY OF CHANGES

Nineteen (19) changes to the Quad Cities Station Unit 2 Technical Specifications have been identified as a result of the proposed license amendment. Two (2) are related to the drywell temperature instrumentation relocation and seventeen (17) correct existing typographical errors. The changes are listed below as follows:

1) Page 3.2/4.2-15 (DPR-30)

- (a) Under column marked "Instrument Readout Location Unit 2", for Parameter titled "Drywell temperature", replace "902-21" with "902-3".

This change reflects the fact that the drywell temperature indication has been moved from a back panel to a front panel location. Such a change where an instrument location is being revised, is administrative in nature.

2) Page 3.2/4.2-18, Table 4.2-2, (DPR-30)

- (a) Under column marked "Instrument Readout Location Unit 1", replace "Unit 1" with "Unit 2".

This change corrects a typographical error (Unit 1 when it should be Unit 2 since it pertains to DPR-30). This is an administrative change.

- (b) Under column marked "Instrument Readout Location Unit 1", (which was changed to "Instrument Readout Location Unit 2" as described in Item 2(a)), for all parameters starting with reactor pressure through Drywell pressure inclusive, change each "901" to "902". There are eight (8) such individual changes where "901" should be replaced with "902".

These changes are being made to reflect the fact that panels are Unit 2 panels not Unit 1 panels, hence 902 instead of 901. These changes will correct typographical errors that exist in the current Quad Cities Unit 2 Technical Specifications (DPR-30) and is administrative.

- (c) Under column marked "Instrument Readout Location Unit 1", (which was changed to "Instrument Readout Location Unit 2", as described in Item 2(a)), for parameter titled "Drywell temperature", replace "901-21" with "902-3".

This change reflects the fact that the drywell temperature indication has been moved from a back panel to a front panel location (identical to the change described in 1(a)). Such a change, where an instrument location is being revised, is administrative in nature.

- (d) Under column marked "Instrument Readout Location Unit 1", (which was changed to "Instrument Readout Location Unit 2", as described in Item 2(a)), for all parameters starting with Neutron monitoring through Main Steam RV position, temperature monitor inclusive, change each "901" to "902". There are five (5) such individual changes where "901" should be replaced with "902".

These changes are being made to reflect the fact that panels are Unit 2 panels not Unit 1 panels, hence 902 instead of 901. These changes will correct typographical errors that exist in the current Quad Cities Unit 2 Technical Specifications (DPR-30) and is administrative.

3) Page 3.2/4.2-13a (DPR-30)

- (a) Same change as described in Item 2(a), under column marked "Instrument Readout Location Unit", replace "Unit 1" with "Unit 2".

This change is an administrative change.

- (b) Same change as described in Item 2(b). Replace the Instrument Readout Location for the Main Steam SV Position Acoustic Monitor and Main Steam SV Position, Temperature Monitor from "901-21" to "902-21".

This change is made in two (2) places on Table 4.2.2. These changes are administrative changes.

### ATTACHMENT 3

#### EVALUATION OF SIGNIFICANT HAZARDS CONSIDERATION AND DESCRIPTION OF PROPOSED AMENDMENT REQUEST

There are two types of changes associated with this proposed license amendment -- both of which are administrative in nature. The first type of change results from the completion of a Unit 2 Detailed Control Room Design Review (DCRDR) Human Factors modification which resulted in the relocation of the drywell temperature indicator from the 902-21 (back) panel to the 902-3 (front) panel. Two (2) such changes need to be incorporated into Tables 3.2-4 and 4.2-2 of LPR-30. The remaining seventeen (17) changes correct typographical errors in the existing Technical Specifications.

#### BASIS FOR NOT SIGNIFICANT HAZARDS CONSIDERATION

Commonwealth Edison has evaluated this proposed amendment and determined that it involves no significant hazards consideration. In accordance with the criteria of 10 CFR 50.92(c), a proposed amendment to an operating license involves no significant hazards considerations if operation of the facility, in accordance with the proposed amendment, would not:

- 1) Involve a significant increase in the probability or consequences of an accident previously evaluated because:
  - (a) The location of the drywell temperature indication has been relocated from the back to the front panel in the Control Room which is an enhancement over the past previous location which was approved by the NRC. This is considered to be a change in the conservative direction and therefore, does not result in an increase in the probability or consequences of previously evaluated accidents.
  - (b) The other changes merely correct typographical errors which have existed in the current Quad Cities Station Unit 2 Technical Specifications (DPR-30). The correction of typographical errors is considered to be an administrative change which does not affect the probability or consequences of previously evaluated accidents.
  
- 2) Create the possibility of a new or different kind of accident from any accident previously evaluated because:
  - (a) There are no hardware changes (addition or deletion of equipment) nor are there any new modes of operation associated with this amendment. The changes to Tables 3.2-4 and 4.2-4 reflect changes to equipment (instrumentation) location only. Therefore, the proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

- (b) The correction of typographical errors does not introduce any new equipment or modes of operation at Quad Cities Station that would create the possibility of a new or different kind of accident than that which was previously evaluated.
- 3) Involve a significant reduction in the margin of safety because:
- (a) Revising an instrument location readout in the control room does not adversely affect the operation of any plant systems. Therefore, the margin of safety has not been unchanged as a result of this change.
  - (b) The correction of typographical errors is considered to be administrative in nature. There are no changes being made to hardware in the manner that plant system are being operated as a result of this license amendment. Therefore, the margin of safety is not being compromised as a result of this administrative change.

Therefore, since the proposed license amendment satisfies the criteria specified in 10 CFR 50.92, Commonwealth Edison has determined that a no significant hazards consideration exist for these items. We further request their approval in accordance with the provisions of 10 CFR 50.91(a)(4).