



**Florida  
Power**  
CORPORATION

July 13, 1988  
3F0788-05

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: Crystal River Unit 3  
Docket No. 50-302  
Operating License No. DPR-72  
Containment Leakage Test Pressure

Dear Sir:

Enclosed is a report concerning the Containment Leakage Test Pressure (Pa) specified in the Crystal River Unit 3 Technical Specifications. This report is provided voluntarily.

Should there be any questions, please contact this office.

Sincerely,

K.R. Wilson, Manager  
Nuclear Licensing

KRW/DGG/dhd  
Enclosure

xc: Regional Administrator, Region II  
Senior Resident Inspector

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*Approved*  
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During a review of the bases for Crystal River Unit 3 Technical Specifications, a discrepancy between the containment leakage test pressure and the maximum containment accident pressure was identified. This review was being performed as part of an effort to upgrade the Technical Specification Bases for Technical Specification improvement in accordance with the NRC Policy Statement on this subject (52FR3708 dated 1/6/87). After a review of other available reference sources, it appears that this discrepancy may apply to several plants with Standard Technical Specifications.

Appendix J of 10CFR50 requires containment leakage testing to be performed at the calculated peak containment internal pressure related to the design basis event and this value be specified either in the Technical Specification or associated bases. The calculated peak containment internal pressure for CR-3 results from a LOCA. The initial containment pressure for this calculation was 14.7 psia (0 psig) and results in a pressure of 49.6 psig. This pressure value is utilized in Technical Specifications 3.6.1.2 (Containment Leakage) and 3.6.1.3 (Containment Air Locks). The containment leakage surveillances are performed in compliance with these Technical Specifications.

In Technical Specification 3.6.1.4 (Internal Pressure), the maximum limit is 17.7 psia (3 psig). The Bases to this Technical Specification indicates the maximum peak pressure obtained from a LOCA event is 49.6 psig. However, it also states that the limit of 3 psig for initial positive containment pressure will assure the total containment pressure does not exceed 52.6 psig. This value is less than the design pressure of containment. This value was confirmed during the Technical Specification upgrade effort.

Additionally, the bases for Technical Specification 3.6.1.6 (Containment Structural Integrity) indicates structural integrity is required to ensure containment will withstand the maximum pressure of 52.6 psig in the event of a LOCA. It also indicates the 10CFR50, Appendix J, Type A leakage test demonstrates this capability (although Technical Specification Surveillance 4.6.1.2.a indicates Type A tests are to be performed at 49.6 psig).

As a result, to assure the containment leakage test pressure bounds the allowable operating limits of the plant, Technical Specifications will be revised to increase the containment leakage test pressure ( $P_a$ ) to 52.6 psig. Applicable surveillance procedures will be revised to reflect this test pressure prior to their next performance.