



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY
WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 17
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Detroit Edison Company (the licensee) dated January 29, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-43 is hereby amended to read as follows:

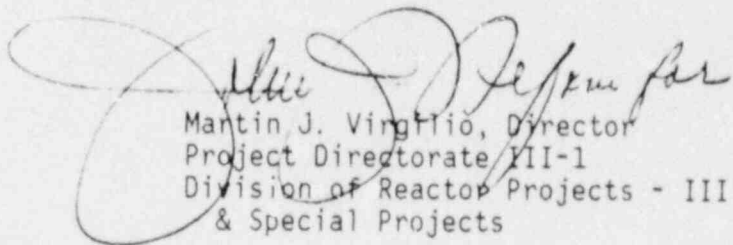
Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 17, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Martin J. Vingilio, Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 29, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 17

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

3/4 6-27
3/4 6-27a
3/4 6-46

INSERT

3/4 6-27
3/4 6-27a
3/4 6-46

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
A. <u>Automatic Isolation Valves</u> ^(a) (Continued)	
14. <u>Group 14 - Drywell and Suppression Pool Ventilation System (Continued)</u>	
Suppression Pool N ₂ and Air Purge Inlet Isolation Valves	
T48-F404	5
T48-F405	5
T48-F409	5
15. <u>Group 15 - Traversing In-core Probe (TIP) System</u>	
Tip System Ball Valves C51-F002 A, B, C, D and E	NA
16. <u>Group 16 - Nitrogen Inerting System</u>	
N ₂ Pressure Control Isolation Valves	
Inboard: T48-F455	60
Outboard: T48-F453	60
T48-F454	60
T48-F456	60
T48-F457	60
T48-F458	60
17. <u>Group 17 - Recirculation Pump System and Primary Containment Radiation Monitoring System</u>	
Recirculation Pumps Seal Purge Isolation Valves	
Inboard: B31-F014A	5
B31-F014B	5
Outboard: B31-F016A	5
B31-F016B	5
Primary Containment Gaseous Radioactivity Monitor Isolation Valves	
Inboard: T50-F450	60
T50-F451	60
Outboard: T50-F455	60
T50-F456	60

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Amendment No. 10, 17

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

MAXIMUM
 ISOLATION TIME
(Seconds)

VALVE FUNCTION AND NUMBER

A. Automatic Isolation Valves^(a) (Continued)

18. Group 18 - Primary Containment Pneumatic Supply System

N₂ to Drywell Isolation Valves

Inboard: T49-F601
 T49-F602

Outboard: T49-F465
 T49-F468

60
 60
 60
 60

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

TABLE NOTATIONS (Continued)

8. Group 8 - Reactor Core Isolation Cooling (RCIC) System
RCIC Steam Line Flow - High
RCIC Steam Supply Pressure - Low
RCIC Turbine Exhaust Diaphragm Pressure - High
RCIC Equipment Room Temperature - High
9. Group 9 - Reactor Core Isolation Cooling (RCIC) Vacuum Breakers
Drywell Pressure - High with simultaneous RCIC
Steam Supply Pressure - Low
10. Group 10 - Reactor Water Cleanup (RWCU) System (Inboard)
RWCU Differential Flow - High
RWCU Area Temperature - High
RWCU Area Ventilation Differential Temperature - High
Reactor Vessel Low Water Level - Level 2
NRHX Outlet Temperature - High
11. Group 11 - Reactor Water Cleanup (RWCU) System (Outboard)
SLCS Initiation (not a containment isolation signal)
RWCU Differential Flow - High
RWCU Area Temperature - High
RWCU Area Ventilation Differential Temperature - High
Reactor Vessel Low Water Level - Level 2
NRHX Outlet Temperature - High
12. Group 12 - Torus Water Management System (TWMS)
Reactor Vessel Low Water Level - Level 2
Drywell Pressure - High
13. Group 13 - Drywell Sumps
Reactor Vessel Low Water Level - Level 3
Drywell Pressure - High
14. Group 14 - Drywell and Suppression Pool Ventilation System
Reactor Vessel Low Water Level - Level 2
Drywell Pressure - High
Fuel Pool Ventilation Exhaust Radiation - High
15. Group 15 - Traversing In-Core (TIP) System
Reactor Vessel Low Water Level - Level 3
Drywell Pressure - High

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
B. <u>Remote-Manual Isolation Valves</u>^(e)	
1. <u>Main Steam Isolation Valves (MSIV) Leakage Control Valves</u> B21-F434	NA
2. <u>RHR Shutdown Cooling Suction Inboard Isolation Valve Bypass Valve</u> ^(q) E11-F608	NA
3. <u>LPCI Inboard Isolation Valves</u> ^(f) Loop A: E11-F015A E11-F610A Loop B: E11-F015B E11-F610B	NA
4. <u>RHR Pumps Recirculation Motor Operated Valves</u> ^{(b)(g)} Pumps A/C: E11-F007A Pumps B/D: E11-F007B	NA
5. <u>Warmup and Flush Line Isolation Valve</u> ^(b) E11-F026B	NA
6. <u>Reactor Protection System Instrumentation Isolation Valves</u> Division I: E11-F412 E11-F413 Division II: E11-F414 E11-F415	NA
7. <u>RHR Pump Torus Suction Isolation Valves</u> ^(b) Pump A: E11-F004A Pump B: E11-F004B Pump C: E11-F004C Pump D: E11-F004D	NA