

APPENDIX A

NOTICE OF DEVIATION

Power Authority of New York
James A. FitzPatrick Nuclear Power Plant

Docket No. 50-333
License No. DPR-59

During an NRC inspection conducted on May 16-20, 1988, a deviation was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C, (Enforcement Policy 1988), the deviation is listed below:

NUREG-0737, Supplement 1, Part 8, Emergency Response Facilities, states, in part, that the TSC will be:

1. Environmentally controlled to provide room air temperature, humidity and cleanliness appropriate for personnel and equipment; and
2. Provided with radiological protection and monitoring equipment necessary to assure that radiation exposure to any person working in the TSC would not exceed 5 rem whole body, or its equivalent to any part of the body, for the duration of the accident.

Contrary to the above, the TSC HVAC system was inoperative in the Emergency mode and the study (Deems Report) performed to determine radiological habitability was inadequate in that the study did not provide information to determine if adequate shielding is available. Specifically: the response only considered radiation from plant systems under accident conditions and did not consider radiation from the radioactive plume during accidental airborne releases; no supportive data was provided for conclusions reached in the study; and no rationale was provided concerning the glass west wall and the add-on shield wall for the condensate storage tank outside the TSC.

Pursuant to the provisions of 10 CFR 2.201, New York Power Authority is hereby required to submit to this office within thirty days of the date of the letter which transmitted this Notice, a written statement or explanation in reply, including: (1) the corrective steps which have been taken and the results achieved; (2) corrective steps which will be taken to avoid further violations; and (3) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending this response time.

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APPENDIX B

APPRAISAL OPEN ITEMS

1. Dose calculation procedures do not contain a method for calculating offsite doses resulting from a real time accident in which the source term is leakage through the drywell boundary and whether any other credible, unmonitored accident release pathways exist for which similar methods should be developed (50-333/88-05-01).
2. The validation and verification documentation for the class A Dose Assessment Model is incomplete (50-333/88-05-02).
3. There is no formalized method for controlling and documenting the maintenance of the dose assessment model (i.e. MMRAS and IDAC) (50-333/88-05-03).

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