Yellow

MAR 31 1988

Docket Nos. 50-369, 50-370
License Nos. NPF-9, NPF-17

Duke Power Company
ATTN: Mr. H. B. Tucker, Vice President
Nuclear Production Department
422 South Church Street
Charlotte, NC 28242

Gentlemen:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION CONCERNING RESPONSE TO BULLETIN 85-03, MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

Duke Power Company's letters of May 16 and November 20, 1986, and February 18, 1987, pertaining to McGuire Units 1 and 2, contained responses to NRC Bulletin 85-03, "Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings." Your letter of January 14, 1988, indicates responses for McGuire will be submitted at the end of fuel cycles 5 (Unit 1) and 4 (Unit 2). The review of these responses by the Nuclear Regulatory Commission indicates the need for additional information, identified by the enclosure, before the program to assure valve operability can be completed.

Please provide the additional information as stated in the enclosure. It is requested that you submit the additional information within 30 days of the date of this letter. Should you have any questions concerning this letter, please contact S. Tingen at (404) 331-2603.

Sincerely,

Jerome Blake
signed original

Alan R. Herdt, Chief
Engineering Branch
Division of Reactor Safety

Enclosure:
Request for Additional Information

cc w/encl:
T. L. McConnell, Station Manager
Senior Resident Inspector - Catawba

bcc w/encl:
B. Bonser, RII
D. Hood, NRR
R. Kiessel, NRR
NRC Resident Inspector
DRS Technical Assistant
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REQUEST FOR ADDITIONAL INFORMATION REGARDING
RESPONSES TO ACTION ITEM E OF NRC BULLETIN 85-03
MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

By letters dated May 16 and November 20, 1986 and February 18, 1987 you responded to NRC Bulletin 85-03, "Motor-Operated Valve Common Mode Failure During Plant Transients Due to Improper Switch Settings" for McGuire Nuclear Station, Units 1 and 2. Provide the following additional information regarding Action Item e of NRC Bulletin 85-03:

1. Clarify whether water hammer due to valve closure has been considered in the determination of pressure differentials. If not, explain.

2. The following MOVs of the SI System are not included in the response of 05-16-86; however, they are shown in the WOG Report of March 1986. Revise the response to include these MOVs, or justify their exclusion. As required by Action Item a of the bulletin, assume inadvertent equipment operations.

   (a) MOVs NI144B, NI115B and NI147A are shown normally open in the SI pump miniflow lines in zones G-9, H-9, and G-11 of Drawing MC-1562-3.0 Revision 9 (Unit 1). They are shown on Page 25 of the WOG Report, as HV-8814B, HV-8814A and HV-8813, respectively.

   NOTE: Similarly located valves are used for Unit 2 also.

   (b) MOVs NV150B and NV151A are shown normally open in the CCP miniflow lines on FSAR Figure 9.3.4-2 (Unit 1). They are shown on Page 24 of the WOG Report, as HV-8111 and HV-8110, respectively.

   NOTE: Similarly located valves are use for Unit 2 also.

3. The following MOVs in the AFW System are not included in the response of 05-16-86. Explain this exception to the Westinghouse recommendation that "all MOVs within the AFW system should be included on the list of valves to be examined for maximum differential pressure", as stated on Page 5 of the WOG Report. Revise the response of 05-15-86 to include these MOVs, or justify their exclusion.

   (a) MOVs CA162C and CA161C are shown normally closed in series in the AFW suction line from Nuclear Service Water Header 1A in zones D-7 and D-8 of Drawing MC-1592-1.1 Revision 6 (Unit 1).

   (b) MOV CA6 is shown normally open in the AFW suction line from the AUX FDW Condensate Storage Tank in Zone C-10 of Drawing MC-1592-1.1 Revision 6 (Unit 1).

(c) MOV CA4 is shown normally open in the AFW suction line from upper surge tanks 1A and 1B in Zone D-9 of Drawing MC-1592-1.1 Revision 6 (Unit 1).

Note: Similarly located valves are used for Unit 2 also.

4. Clarify the response of 05-15-86 to indicate whether the tabulated differential pressure apply to opening the valve, closing the valve or both opening and closing.

5. Regarding the proposed program for action items b, c and d of the bulletin, provide the following as a minimum:

(a) a commitment to justify continued operation of a valve determined to be inoperable, and

(b) consideration of pipe break conditions as required by the bulletin.