



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 113
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Virginia Electric and Power Company (the licensee) dated April 23, 1987 and May 29, 1987, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

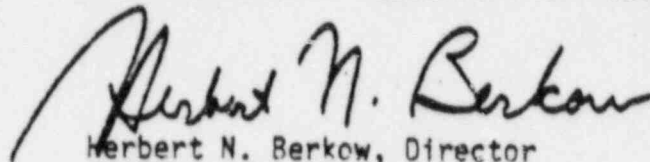
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(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, and shall be implemented within 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 15, 1988



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMFNDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. DPR-37

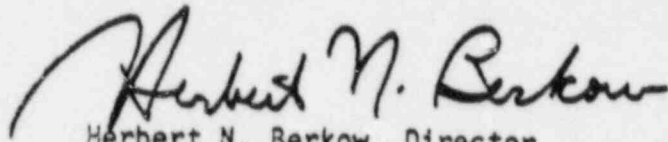
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Virginia Electric and Power Company (the licensee) dated April 23, 1987 and May 29, 1987, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, and shall be implemented within 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 15, 1988

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 118 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 118 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS 3.7-21
TS 3.21-7
TS 4.1-9a

Insert Pages

TS 3.7-21
TS 3.21-7
TS 4.1-9a

TABLE 3.7-6

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Auxiliary Feedwater Flow Rate	1 per S/G	1 per S/G
2. Inadequate Core Cooling Monitor		
a. Reactor Vessel Coolant Level Monitor	2	1
b. Reactor Coolant System Subcooling Margin Monitor	2	1
c. Core Exit Thermocouples	2 (Note 2)	1 (Note 2)
3. PORV Position Indicator (Primary Detector)	1/valve	1/valve
4. PORV Position Indicator (Backup Detector)	1/valve	0
5. PORV Block Valve Position Indicator	1/valve	1/valve
6. Safety Valve Position Indicator (Primary Detector)	1/valve	1/valve
7. Safety Valve Position Indicator (Backup Detector)	1/valve	0
8. Containment Pressure	2	1
9. Containment Water Level (Narrow Range)	2	1
10. Containment Water Level (Wide Range)	2	1
11. Containment High Range Radiation Monitor	2	1 (Note 1, b and c only)
12. Process Vent High Range Effluent Monitor	2	2 (Note 1, a, b, and c)
13. Ventilation Vent High Range Effluent Monitor	2	2 (Note 1, a, b, and c)
14. Main Steam High Range Radiation Monitors (Units 1 and 2)	3	3 (Note 1, a, b, and c)
15. Aux. Feed Pump Steam Turbine Exhaust Radiation Monitor	1	1 (Note 1, a, b, and c)

Note 1: With the number of operable channels less than required by the Minimum Channels Operable requirements

- Initiate the preplanned alternate method of monitoring the appropriate parameter(s), within 72 hours
- Either restore the inoperable channel to operable status within 7 days of the event, or
- Prepare and submit a Special Report to the commission pursuant to specification 6.2 within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable.

Note 2: A minimum of 2 core exit thermocouples per quadrant is required for the channel to be operable.

TABLE 3.21-1
FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>	
	<u>Heat</u>	<u>Smoke</u>
1. Containment (Reactor Coolant Pumps Only)***	1 per RCP	-
2. Containment Cable Penetration Area		4
3. Containment Recirculation Air System		1
4. Cable Tray Room	3	4
5. Cable Tunnel	2	3
6. Cable Vault Area		
Lower Area	1	2
Upper Area	1	1
7. Charcoal Filter Banks	1 per bank	-
8. Emergency Diesel Generator Room	1 per room	-
9. Fuel Oil Tank Room (river)	1**	1
10. Fuel Oil Transfer Pump Houses	1 per house**	-
11. Control Room		4
12. Emergency Switchgear Room		3
13. Auxiliary Building General Area		14
14. Auxiliary Building Charging Pump Cubicles		1 per cubicle
15. Main Steam Valve House		3
16. Safeguards Area		1
17. Fuel Building		1
18. Fire Pump Building		2
19. Mechanical Equipment Room #3		1
20. Battery Room 1A		1
21. Battery Room 1B		1
22. Battery Room 2A		1
23. Battery Room 2B		1

** Rate of rise actuation devices for high pressure CO₂ system

*** One heat detector installed per pump. RCP pump bearing and motor temperature will be monitored once per hour if the RCP heat detector is inoperable.

TABLE 4.1-2

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Auxiliary Feedwater Flow Rate	P	R
2. Inadequate Core Cooling Monitor	M	R
3. PORV Position Indicator (Primary Detector)	M	R
4. PORV Position Indicator (Backup Detector)	M	R
5. PORV Block Valve Position Indicator	M	R
6. Safety Valve Position Indicator	M	R
7. Safety Valve Position Indicator (Backup Detector)	M	R
8. Containment Pressure	M	R
9. Containment Water Level (Narrow Range)	M	R
10. Containment Water Level (Wide Range)	M	R

M = Monthly

P = Prior to each startup if not done within the previous week

R = Refueling