Docket No.: 50-425

Mr. W. G. Wainston, III

Mr. W. G. Hairston, III Senior Vice President -Nuclear Operations Georgia Power Company P. O. Box 4545 Atlanta, Georgia 30302

Dear Mr. Hairston:

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Docket File VOGTLE PLANT FILE

EJordan MNBB-3302

OGC Local PDR MRC PDR

PDII-3 r/f JHopkins

MRood DMatthews

SVarga 14E-4 GLainas 14H-3

BGrimes 9A-2

ACRS (10)

SUBJECT: VOGTLE UNIT 2 REQUEST FOR ADDITIONAL INFORMATION SPENT FUEL RACKS (TAC 67079)

The NRC staff is reviewing the Vogtle Unit 2 spent fuel racks described in letters submitted December 23, 1987, and May 5, 1988. The submittals do not address actions concerning the potential consequences of three postulated design basis accidents. These accidents are the fuel handling, cask drop, and cask transport accidents. The radiological consequences of these accidents were previously analyzed by the staff and reported in the Safety Evaluation Report related to the operation of Vogtle Units 1 and 2. The previous fuel handling and cask drop accidents for Vogtle Unit 1 spent fuel racks do not require reevaluation because the operations potentially involved with these accidents are not modified. Vogtle Unit 2 spent fuel racks require this reevaluation because the spent fuel racks are designed differently from the ones currently in use for Vogtle Unit 1. Therefore, the NRC staff finds that it needs additional information (see enclosure) in order to complete its review. In order to maintain our review schedule, we request a response by August 1, 1988.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than 10 respondents; therefore, OMB clearance is not required under P. L. 96-511.

Sincerely,

Jon B. Hopkins, Project Manager Project Directorate II-3 Division of Reactor Projects I/II

Enclosure: As stated

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cc: See next page

PDII-3 MRbod 6/2 988 PDII-3 JHopkins:sw 6/29/88 PM1-3 DMatthews

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REQUEST FOR ADDITIONAL INFORMATION VOGTLE UNIT 2 SPENT FUEL RACKS

470 #6

Perform an analysis of the radiological consequences of postulated fuel handling accident (NUREG-0800, "Standard Review Plan," Section 15.7.4) and spent fuel cask drop accidents (NUREG-0800, "Standard Review Plan," Section 15.7.5) inside the containment.

470 #7

If changed from previous assessment provide the number of spent fue? assemblies that could be damaged by dropping each typical load carried over the pool.

Mr. W. G. Hairston, III Georgia Power Company

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