



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-366
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated January 4, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 91, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

Kahtan N. Jabbur, Acting Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 12, 1988

PD#II-3/DRP-I/II
MRood/mac
01/30/88
AD:R.II/DRP-I/II
GLainas

mc
PD#II-3/DRP-I/II
LCrocker
01/20/88

msd
NRR-SRXB
Wlodges
01/21/88

OGC-WFlint
~~01/1/88~~
03/9/88

KNJ
PD#II-3/DRP-I/II
KJabbur
03/9/88

Janice Moore
checked - see
cover sheet. j/k

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u> <u>Page</u>	<u>Insert</u> <u>Page</u>
1-11	1-11
3/4 3-12	3/4 3-12
3/4 3-15	3/4 3-15
3/4 6-36	3/4 6-36
3/4 6-37	3/4 6-37
3/4 6-40	3/4 6-40
3/4 7-1	3/4 7-1
3/4 7-2	3/4 7-2

TABLE 1.2
OPERATIONAL CONDITIONS

<u>CONDITION</u>	<u>MODE SWITCH POSITION</u>	<u>AVERAGE REACTOR COOLANT TEMPERATURE</u>
1. POWER OPERATION	Run	Any Temperature
2. STARTUP	Startup/Hot Standby	Any Temperature
3. HOT SHUTDOWN	Shutdown	> 212°F***
4. COLD SHUTDOWN	Shutdown	≤ 212°F***
5. REFUELING*	Refuel**	≤ 212°F

* Reactor vessel head unbolted or removed and fuel in the vessel.

** See Special Test Exception 3.10.3.

*** During the performance of inservice hydrostatic or leak testing with all control rods fully inserted and reactor coolant temperature above 212°F, the reactor may be considered to be in the COLD SHUTDOWN condition for the purpose of determining Limiting Condition for Operation applicability. However, compliance with an ACTION requiring COLD SHUTDOWN shall require a reactor coolant temperature ≤ 212°F. In addition, compliance with the following Specifications is required when performing the hydrostatic and leak testing under the identified conditions: 3.6.5.1, 3.6.5.2, 3.6.6.1, and 3.7.1.1.

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL(a)	MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)	APPLICABLE OPERATIONAL CONDITION	ACTION
2. SECONDARY CONTAINMENT ISOLATION				
a. Reactor Building Exhaust Radiation - High (2D11-K609 A, B, C, D)	6, 10, 12, *	2	1, 2, 3, 5, ** and ***	24
b. Drywell Pressure - High (2C71-N650 A, B, C, D)	2, 6, 7, 10, 12, *	2	1, 2, 3	24
c. Reactor Vessel Water Level - Low Low (Level 2) (2B21-N682 A, B, C, D)	5, *	2	1, 2, 3 and ***	24
d. Refueling Floor Exhaust Radiation - High (2D11-K611 A, B, C, D)	6, 10, 12, *	2	1, 2, 3, 5, ** and ***	24
3. REACTOR WATER CLEANUP SYSTEM ISOLATION				
a. Δ Flow - High (2G31-N603 A, B)	5	1	1, 2, 3	25
b. Area Temperature - High (2G31-N662 A, D, E, H, J, M)	5	1	1, 2, 3	25
c. Area Ventilation Δ Temp. - High (2G31-N663 A, D, E, H, J, M; 2G31-N661 A, D, E, H, J, M; 2G31-N662 A, D, E, H, J, M)	5	1	1, 2, 3	25
d. SLCS Initiation (NA)	5(*)	NA	1, 2, 3	25
e. Reactor Vessel Water Level - Low Low (Level 2) (2B21-N682 A, B, C, D)	5, *	2	1, 2, 3	25

Hatch - Unit 2

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Amendment No. 91

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATIONACTION

- ACTION 20 - Be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 21 - Be in at least STARTUP with the main steam line isolation valves closed within 2 hours or be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 22 - Be in at least STARTUP within 2 hours.
- ACTION 23 - Be in at least STARTUP with the Group 1 isolation valves closed within 2 hours or in at least HOT SHUTDOWN within 6 hours.
- ACTION 24 - Establish SECONDARY CONTAINMENT INTEGRITY with the standby gas treatment system operating within one hour.
- ACTION 25 - Isolate the reactor-water cleanup system.
- ACTION 26 - Close the affected system isolation valves and declare the affected system inoperable.
- ACTION 27 - Verify power availability to the bus at least once per 12 hours or close the affected system isolation valves and declare the affected system inoperable.
- ACTION 28 - Close the shutdown cooling supply and reactor vessel head spray isolation valves unless reactor steam dome pressure \leq 145 psig.
- ACTION 29 - Either close the affected isolation valves within 24 hours or be in hot shutdown within the next 6 hours and in cold shutdown within the next 30 hours.

NOTES

- * Actuates the standby gas treatment system.
- ** When handling irradiated fuel in the secondary containment.
- *** When performing inservice hydrostatic or leak testing with the reactor coolant temperature above 212°F.
- a. See Specification 3.6.3, Table 3.6.3-1 for valves in each valve group.
- b. A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- c. With a design providing only one channel per trip system, an inoperable channel need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, the inoperable channel shall be restored to OPERABLE status within 2 hours or the ACTION required by Table 3.3.2-1 for that Trip Function shall be taken.
- d. Trips the mechanical vacuum pumps.
- e. A channel is OPERABLE if 2 of 4 instruments in that channel are OPERABLE.
- f. May be bypassed with all turbine stop valves closed.
- g. Closes only RWCU outlet isolation valve 2G31-F004.
- h. Alarm only.
- i. Adjustable up to 60 minutes.
- j. Isolates containment purge and vent valves.

CONTAINMENT SYSTEMS

3/4.6.5 SECONDARY CONTAINMENT

SECONDARY CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.5.1 Hatch-Unit 2 SECONDARY CONTAINMENT INTEGRITY and Hatch-Unit 1 secondary containment integrity shall be maintained.

APPLICABILITY: CONDITIONS 1, 2, 3, and *.

ACTION:

Without Hatch-Unit 2 SECONDARY CONTAINMENT INTEGRITY and/or without Hatch-Unit 1 secondary containment integrity, restore Hatch-Unit 2 SECONDARY CONTAINMENT INTEGRITY and Hatch-Unit 1 secondary containment integrity within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.5.1.1 Hatch-Unit 2 SECONDARY CONTAINMENT INTEGRITY shall be demonstrated by:

- a. Verifying at least once per 31 days:
 1. All equipment hatches are closed and sealed, and
 2. At least one door in each access to the secondary containment is closed.
- b. Verifying at least once per 92 days that each secondary containment ventilation system automatic isolation damper is OPERABLE or secured in the closed position per Specification 3.6.5.2.
- c. At least once per 18 months:
 1. Verifying that one standby gas treatment subsystem will draw down the secondary containment to $\geq 1/4$ inch of vacuum water gauge in ≤ 120 seconds, and
 2. Operating one standby gas treatment subsystem for one hour and maintaining $\geq 1/4$ inch of vacuum water gauge in the secondary containment at a flow rate not exceeding 4000 CFM.

4.6.5.1.2 Hatch-Unit 1 secondary containment integrity shall be demonstrated per Hatch-Unit 1 Technical Specifications.

*When performing inservice hydrostatic or leak testing with the reactor coolant temperature above 212°F.

CONTAINMENT SYSTEMS

SECONDARY CONTAINMENT AUTOMATIC ISOLATION DAMPERS

LIMITING CONDITION FOR OPERATION

3.6.5.2 The secondary containment ventilation system automatic isolation dampers shown in Table 3.6.5.2-1 shall be OPERABLE.

APPLICABILITY: CONDITIONS 1, 2, 3, and *.

ACTION:

With one or more of the secondary containment ventilation system automatic isolation dampers specified in Table 3.6.5.2-1 inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable, provided that at least one isolation damper is maintained OPERABLE in each affected penetration that is open, and;

- a. The inoperable damper is restored to OPERABLE status within 8 hours, or
- b. The affected penetration is isolated by use of a closed damper within 8 hours, or
- c. SECONDARY CONTAINMENT INTEGRITY is demonstrated within 8 hours and the damper is restored to OPERABLE status within 7 days.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours, or

SURVEILLANCE REQUIREMENTS

4.6.5.2 Each secondary containment ventilation system automatic isolation damper specified in Table 3.6.5.2-1 shall be demonstrated OPERABLE:

1. At least once per 92 days by cycling each automatic damper testable during plant operation through at least one complete cycle of full travel.

*When performing inservice hydrostatic or leak testing with the reactor coolant temperature above 212°F.

CONTAINMENT SYSTEMS

3/4.6.6 CONTAINMENT ATMOSPHERE CONTROL

STANDBY GAS TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.6.1 Two Hatch-Unit 2 independent standby gas treatment subsystems and two Hatch-Unit 1 independent standby gas treatment subsystems shall be OPERABLE.

APPLICABILITY: CONDITIONS 1, 2, 3, and *.

ACTION:

- a. With one of the above required standby gas treatment subsystems inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With two or more of the above required standby gas treatment subsystems inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.6.1.1 Each Hatch-Unit 2 standby gas treatment subsystem shall be demonstrated OPERABLE:

- a. By initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least a total of 10 hours each 31 days with the heaters on automatic control.
- b. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the system by:
 1. Verifying that the cleanup system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 1, July 1976, and the system flow rate is $4000 \pm 0, -1000$ cfm.
 2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.

*When performing inservice hydrostatic or leak testing with the reactor coolant temperature above 212°F.

3/4.7 PLANT SYSTEMS

3/4.7.1 SERVICE WATER SYSTEMS

RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.1 Two independent Residual Heat Removal Service Water (RHRSW) System subsystems shall be OPERABLE with each subsystem comprised of:

- a. Two OPERABLE pumps, and
- b. An OPERABLE flow path for heat removal capable of taking suction from the intake and transferring the water through an RHR heat exchanger.

APPLICABILITY: CONDITIONS 1, 2, 3, 5, and *.

ACTION:

- a. In CONDITION 1, 2 or 3:
 1. With one RHRSW pump inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore the inoperable pump to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 2. With one RHRSW pump in each subsystem inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore at least one inoperable pump to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 3. With one RHRSW subsystem inoperable, restore the inoperable subsystem to OPERABLE status with at least one OPERABLE pump within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 4. With both RHRSW subsystems inoperable, restore at least one subsystem to OPERABLE status within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and either be in COLD SHUTDOWN or maintain the reactor coolant system temperature < 400°F by use of alternate heat removal methods within the following 24 hours.

*When performing inservice hydrostatic or leak testing with the reactor coolant temperature above 212°F.

PLANT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

- b. In CONDITION 5, with up to 3 RHRSW pumps inoperable or with one RHRSW subsystem inoperable, restore both subsystems with at least one pump in each system to OPERABLE status within 7 days or declare the RHR system inoperable for decay heat removal and reactor coolant circulation and take the ACTION required by Specification 3.9.12.
- c. In condition *, with up to 3 RHRSW pumps inoperable or with one RHRSW subsystem inoperable, restore both subsystems with at least one pump in each system to OPERABLE status within 7 days or be in COLD SHUTDOWN within the next 24 hours.

SURVEILLANCE REQUIREMENTS

4.7.1.1 Each residual heat removal service water system subsystem shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position, and
- b. At least once per 92 days by verifying that each pump develops a discharge pressure of ≥ 300 psig at a flow of ≥ 4000 gpm.