LICENSEE EVENT REPORT

CONTROL BLOCK: [| | | | |] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 — Licensee Code (7–14)	M A P P S 1 (2)
License Number (15–25)	0 0 0 0 0 0 0 0 0 0 0 (3)
License Type (26–30)	4 1 1 1 1 (4)
CAT (57–58)	(5)

CON'T

0 1 — Report Source (60)	L (6)
Docket Number (61–68)	0 5 0 - 0 2 9 3 (7)
Event Date (69–74)	1 0 0 9 7 8 (8)
Report Date (75–80)	1 1 0 7 7 8 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On October 9, 1978 the reactor scrammed on a Load Rejection caused by a ground in the
0 3 potential indicating circuit for the Aux. transformer which activated the Differential
0 4 Overcurrent Relay Logic resulting in a Generator Lockout Relay trip. Subsequent to the
0 5 scram, RV-203-3C opened and did not reseat until pressure had decreased to approximately
0 6 330 PSIG. During the ensuing cooldown, RV-203-3C opened and closed a second time. The
0 7 cooldown rate did not exceed that allowed by Tech. Spec. 3.6.A.1.
0 8

0 9	System Code (11)	Cause Code (12)	Cause Subcode (13)	Component Code (14)	Comp. Subcode (15)	Valve Subcode (16)
	C C	E	B	V A L V E X	F	B

(17) LER/RO Report Number	Event Year		Sequential Report No.		Occurrence Code	Report Type		Revision No.
	7 8	—	0 4 8	/	0 3	L	—	0

Action Taken (18)	Future Action (19)	Effect on Plant (20)	Shutdown Method (21)	Hours (22)	Attachment Submitted (23)	NPRD-4 Form Sub. (24)	Prime Comp. Supplier (25)	Component Manufacturer (26)
A	F	A	C	0 4 3 8	Y	Y	N	T 0 2 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this event was excessive pilot seat leakage which pressurized the second
1 1 stage piston chamber keeping the second stage and main valve open. The relief/safety
1 2 valve was replaced in kind and new design top works have been ordered.
1 3
1 4

1 5	Facility Status (28)	% Power (29)	Other Status (30)	Method of Discovery (31)	Discovery Description (32)
	E	1 0 0	N.A.		Operat[illegible]al Event

1 6	Activity Released (33)	Content of Release (34)	Amount of Activity (35)	Location of Release (36)
	Z	Z	N.A.	N.A.

PERSONNEL EXPOSURES

1 7	Number (37)	Type (38)	Description (39)
	0 0 0	Z	N.A.

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PERSONNEL INJURIES

1 8	Number (40)	Description (41)
	0 0 0	N.A.

LOSS OF OR DAMAGE TO FACILITY (43)

1 9	Type (42)	Description
	Z	N.A.

PUBLICITY

2 0	Issued (44)	Description (45)	NRC USE ONLY
	N	N.A.	

NAME OF PREPARER Charles J. Mathis PHONE: 617-746-7900-106

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Attachment to LER 78-048/03L-0

Event Description

On October 9, 1978, at 2121 hours the reactor scrammed on a Load Rejection caused by a trip of the Generator Lockout Relay. The trip occurred as a result of a Nuclear Plant Operator attempting to change a burned out bulb for Auxiliary Transformer phase indication on Panel C-3. The bulb's socket broke and grounded to the panel causing a current surge which activated the Differential Over-current Relay Logic.

Subsequent to the scram, it was observed that relief/safety valves RV-203-3A and 3C had opened and one or both had not reclosed. The Watch Engineer ordered TORUS cooling initiated and relief valves 3A & 3C cycled in accordance with station procedure 2.4.28 (Relief/Safety Valve Malfunction). A definite noise and pressure change was observed when RV-203-3A was cycled, however, there was no change noted when RV-203-3C was cycled. Approximately 20 minutes after the scram, with reactor pressure around 330 PSIG, RV-203-3C closed. The cooldown rate during the 20 minute period as indicated on recorder 42-TR-151 A&B, was 294°F/hr.

After RV-203-3C closed, pressure and temperature were allowed to increase to minimize the effects of the blowdown. Approximately 35 minutes later, with reactor pressure about 630 PSIG, RV-203-3C opened again. This resulted in another blowdown which again lasted for approximately 20 minutes before RV-203-3C closed at around 200 PSIG. The cooldown rate during this period was 255°F/hr.

The pressure and temperature were allowed to increase again and no further incidents of relief valve opening occurred. Extrapolation of the cooldown rates from 42-TR-151A&B show that the cooldown rate did not exceed the 100°F/hr. limit of Technical Specification 3.6.A.1 when averaged over a one hour period.

The reactor was manuvered to a cold condition for replacement of CV-203-3C. In addition RV-203-3B was replaced as it has exhibited excessive tail pipe temperatures.

The replacement valves operability was verified in accordance with Station Procedure 8.5.6.1 (Manually Opening of Relief Valve) during the startup following this event.