ENCLOSURE

ATTACHMENT 4

Southern California Edison Company

P. O. BOX 128

SAN CLEMENTE, CALIFORNIA 92674-0128

September 27, 1995

WALTER C. MARSH
MANAGER OF NUCLEAR REGULATORY AFFAIRS

TELEPHONE
(714) 368-7501

Mr. J. L. Pellet, Chief, Operations Section
Region IV
U. S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

Dear Mr. Pellet:

Subject: Docket Numbers 50-361 and 50-362
Proposed Examination Outlines
San Onofre Nuclear Generating Station, Units 2 and 3

References: A) Letter from T. P. Gwynn, (NRC), to H. B. Ray, (Edison), dated August 23, 1995, "Meeting on Pilot Examination Development Program."

B) Letter from T. P. Gwynn, (NRC), to H. B. Ray, (Edison), dated August 11, 1995, "Pilot Initial Examination (50-361/95-19; 50-362/95-19)."

In accordance with NRC Pilot Examination Development Program requirements (Reference A), this letter provides Edison's proposed examination outline (Enclosure). The required Examination Security Agreement has been signed and is included in the sealed enclosure. Based on Reference B, and a telephone call with Mr. Michael Murphy, Licensing Examiner, Region IV, on August 29, 1995, Edison understands NRC staff review of the completed and validated examinations will occur at San Onofre the week of November 13, 1995.

In accordance with the physical security requirements discussed in Reference A, Edison requests the enclosed material be withheld from public disclosure until after the examination process is complete.

If you require any additional information, please let me know.

Sincerely,

Walter C. Marsh

cc: (w/o enclosure)
L. J. Callan, Regional Administrator, NRC Region IV
J. E. Dyer, Director, Division of Reactor Projects, Region IV
K. E. Perkins, Jr., Director, Walnut Creek Field Office, NRC Region IV
J. A. Sloan, NRC Senior Resident Inspector, San Onofre Units 2 & 3
M. B. Fields, NRC Project Manager, San Onofre Units 2 and 3

WITHHOLD FROM PUBLIC DISCLOSURE

Examination Level: SRO
Facility: San Onofre Nuclear Generating Station **Week of Examination:** 27 November 1995
Examiner's Name (print):

Administrative Topic/Subject Description	Describe Method of evaluation: 1. ONE Administrative JPM, OR 2. TWO Administrative Questions		
A.1	Estimated Critical Position	J106S 001-010-A4-03	3.9
	Key Control & Safety	194-001-K1-05	3.4
		194-001-K1-12	2.9
A.2	Remove Equipment from Service	J035S 012-000-A4-05	3.6
A.3	Radiation Control	J126S 194-001-A1-16	4.4
A.4	Emergency Classification	J005 000-067-A2-13	4.4

Examiner: _____ **Chief Examiner:** _____

Examination Level: SRO
Facility: San Onofre Nuclear Generating Station **Week of Examination:** 27 November 1995
Examiner's Name (print):

System / JPM	Safety Function	Planned Follow-up Questions: K/A/G // Importance // Description
J036S - Align four finger CEA 001-000-G0-13 3.6	I	001-010-K4-03 // 3.4
J033FS - Rad Monitor Surveillance 073-000-A4-01 3.9	IX	
J048S - Start RCP post Accident 003-000-G0-09 3.4	IV	
J113S - RAS Actuation Verification 006-020-A3-04 4.3	II	
J115S - Transfer Charging Pump Suction post-LOCA 000-011-A1-05 3.9	III	004-000-K1-06 // 3.1
J017 - Secure CEDMCS MG Set 001-000-G0-09 3.6	I	001-000-G0-07 // 3.3 001-010-K6-03 // 3.3
J019 - Perform actions as CRS outside Control Room 000-068-A1-10 3.9	VIII	062-000-G0-07 // 3.2 062-000-A2-01 // 3.9
J117 - Restore Non-qualified Loads from Outside Control Room 062-000-A2-01 3.9	VII	
J120S - Fuel Handling Isolation System Reset 000-060-A1-02 3.1	XI	
J013S - Transfer Shutdown Cooling from Parallel Pump & Heat Exchanger Operation to Single Pump & Heat Exchanger 005-000-A1-01 3.6	IV	

Examiner: _____ **Chief Examiner:** _____

Simulation Facility: SONGS 2/3 Scenario No.: 1

Examiners: _____ Applicants: _____

_____ _____

_____ _____

Initial conditions: Recovered a Dropped CEA 1 hours ago. Circulating Water DP is between 8 and 9 PSID with Screen wash pump in service. Auxiliary Feedwater pump P-504 Out Of Service for Boundary of the week expected return in four hours.

Turnover: _____

Event No.	Malf. No.	Event Type *	Event Description	K/A's
		PEM	P-504 Rack out Screen Wash Pump Auto Start	
1		RO-R BOP-N	Increase Power to 100%	
2	RC09A	RO-I	Reactor Coolant Pump Speed Sensor Failure	035010A203 3.6
3	RD03	RO-C	Dropped CEA	000003A105 4.1
4	RC03	RO-R BOP-R	Reactor Coolant System Leak	000009A204 4.0
5	RC03	RO-M BOP-M	Loss Of Coolant Accident	000011G011 4.5
6	RP01G	BOP-C	Component Coolant Water Pump Auto Start Failure	008030A304 3.9
7	EC08D	RO-C	HPSI pump Over current ground	006000A202 4.3

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor (PEM) Preexisting Malfunctions

Examiner: _____

Chief Examiner: _____

Simulation Facility: SONGS 2/3 Scenario No.: 2

Examiners: _____ Applicants: _____

_____ _____

_____ _____

Initial conditions: Circulating Water DP is between 8 and 9 PSID with Screen wash pump in service. Auxiliary Feedwater pump P-504 Out Of Service for Boundary of the week expected return in four hours.

Turnover: _____

Event No.	Malf. No.	Event Type *	Event Description	K/A's
		PEM	P-504 Rack out Screen Wash Pump Auto Start	
1	SG03E	RO-I	Steam Generator Pressure Transmitter Failure	035010A203 3.6
2	RX05A	RO-I	Pressurizer Level Setpoint Failure	000028A108 3.6
3	FW12	BOP-C	Condensate Pump Sheared Shaft	056000A204 2.8
4	SG02A	RO-R BOP-N	Steam Generator Tube Leak Down power required	000037G008 3.9
5	CV17A	RO-C	Boric Acid Makeup pump failure OC GRD	004000A401 3.9
6	SG02A	RO-M BOP-M	Steam Generator Tube Rupture	000038A132 4.7

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor (PEM) Preexisting Malfunctions

Examiner: _____

Chief Examiner: _____

Simulation Facility: SONGS 2/3 Scenario No.: 3

Examiners: _____ Applicants: _____

_____ _____

_____ _____

Initial conditions: Circulating Water DP is between 8 and 9 PSID with Screen wash pump in service. Auxiliary Feedwater pump P-504 is Out Service for Boundary of the week expected return in four hours.

Turnover: _____

Event No.	Malf. No.	Event Type *	Event Description	K/A's
		PEM	P-504 Rack out Screen Wash Pump Auto Start	
1	CW05C	BOP-C BOP-N RO-R	Loss of Circulating Water Pump P-117 overcurrent ground Downpower required	075000A202 2.7
2	RC10A	RO-I	RCS T cold Temperture Transmitter Failure	002000A103 3.8
3	ED03B	RO-C	Loss of 4160 Emergency Bus	062000A201 3.8
4	ED01	RO-M BOP-M	Loss of Offsite Power	0000562448 4.5
5	FW25	BOP-C	Overspeed trip of P-140	061000A204 3.8

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor (PEM) Preexisting Malfunctions

Examiner: _____

Chief Examiner: _____

Simulation Facility: SONGS 2/3 Scenario No.: 4 BACKUP

Examiners: _____ Applicants: _____

_____ _____

_____ _____

Initial conditions: Circulating Water DP is between 8 and 9 PSID with Screen wash pump in service. Auxiliary Feedwater pump P-504 Out Of Service for Boundary of the week expected return in four hours.

Turnover: _____

Event No.	Malf. No.	Event Type *	Event Description	K/A's
		PEM	P-504 Rack out Screen Wash Pump Auto Start	
1	WD02	BOP-C	Ruptured Waste Gas Decay Tank	071000A203 3.3
2	FW09A	BOP-N RO-R	Feedwater Pump Turbine High Vibration followed by feedwater pump trip	059000A204 3.3
3	RC10A	RO-I	RCS T Cold Transmitter Failure	016000G011 3.4
4	RP01M	RO-C	Containment Spray Pump Auto Start Failure	026000A301 4.5
5	MS03A	RO-M BOP-M	Main Steam Line Break Inside Containment	000040A201 4.7

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor (PEM) Preexisting Malfunctions

Examiner: _____

Chief Examiner: _____

Knowledge and Ability Record Form
PLANT-WIDE GENERIC RESPONSIBILITIES

Facility: SONGS 2/3

WR - Senior Reactor Operator Target: 17% Actual: 17%

K/A	TOPIC	RATING SRO
19 001A101	Ability to obtain and verify control procedure copy.	3.4
194 01A101	Ability to obtain and verify control procedure copy.	3.4
1940 1A102	Ability to execute procedural steps.	3.9
19400 A102	Ability to execute procedural steps.	3.9
194001 \103	Ability to locate and use procedures and station directives related to shift staffing and activities.	3.4
194001A 06	Ability to make accurate, clear and concise logs, records, status boards, and reports.	3.4
194001A1 0	Ability to coordinate personnel activities outside the control room.	3.9
194001A112	Ability to direct personnel activities outside the control room.	4.1
194001A116	Ability to take actions called for in the Facility emergency Plan, including (if required) supporting or acting as the Emergency Coordinator.	4.4
194001K101	Knowledge of how to conduct and verify valve lineups.	3.7
194001K101	Knowledge of how to conduct and verify valve lineups.	3.7
194001K101	Knowledge of how to conduct and verify valve lineups.	3.7
194001K102	Knowledge of tagging and clearance procedures.	4.1
194001K103	Knowledge of 10CFR20 and related facility radiation control requirements.	3.4
194001K110	Knowledge of safety procedures related to caustic solutions.	3.3
194001K114	Knowledge of safety procedures related to confined spaces.	3.6
194001K116	Knowledge of facility protection requirements, including fire brigade and portable fire-fighting equipment usage.	4.2

Knowledge and Ability Record Form
PLANT SYSTEMS

Facility: SONGS 2/3

Group I Plant Systems Target: 19% Actual: 19%

K/A	TOPIC	RATING SRO
001000K105	NIS and RPS	4.4
003000K104	CVCS	2.9
004000K202	Knowledge of bus power supplies to makeup pumps.	3.1
004000K501	Importance of oxygen control in RCS.	3.3
013000A302	Operation of actuated equipment.	4.2
013000K103	CCS	4.1
015000A105	Imbalance (Axial Shape)	3.9
015000A402	NIS Indicators	3.9
017020A402	Temperature values used to determine RCS/RCP operation during Inadequate Core Cooling	4.1
022000G004	Knowledge of system purpose and/or function	3.3
026000G006	Knowledge of bases in technical specifications for limiting conditions for operation and safety limits	3.6
056000K503	Water hammer and methods of prevention	2.6
059000A203	Overfeeding event	3.1
059000K104	S/G water level control system	3.4
061000K302	S/G	4.4
061000K414	AFW automatic isolation	3.7
063000K103	Battery charger and battery	3.5
071000K404	Isolation of waste gas release tanks	3.4
072000A101	Radition Levels	3.6

Knowledge and Ability Record Form
PLANT SYSTEM

Facility: SONGS 2/3

Group II Plant Systems Target: 17% Actual: 17%

K/A	TOPIC	RATING SRO
002000G006	Knowledge of bases in technical specifications limiting conditions for operations and safety limits	3.8
006000K405	Autostart of HPI/LPI/SIP	4.4
010000A107	RCS pressure	3.7
011000K101	CVCS	3.9
012000A206	Failure of RPS Signal to trip the reactor	4.7
012000K607	Knowledge of the applicable performance and design attributes of the following RPS components CPC	3.2
012000K607	Knowledge of the applicable performance and design attributes of the following RPS components CPC	3.2
016000K307	ECCS	3.7
027000G004	Knowledge of system purpose and/or function	3.4
029000K101	Gaseous radiation release monitors	3.7
033000K401	Maintenance of spent fuel level	3.2
055000K301	Main condenser	2.7
062000K410	Uninterruptible AC power sources	3.5
064000A206	Operating unload, lightly loaded, and highly loaded time limit	3.3
064000A406	Manual start, loading , and stopping of the EDG	3.9
086000K301	Shutdown capability with redundant equipment	3.2
103000A205	Emergency containment entry	3.9

Knowledge and Ability Record Form
PLANT SYSTEMS

Facility: SONGS 2/3

Group III Plants Systems Target: 4% Actual: 4%

K/A	TOPIC	RATING SRO
005000K106	ECCS	3.6
041020A302	RCS pressure, RCS temperature, and reactor power	3.4
076000K101	CCW System	3.3
078000K103	Containment air	3.4

EMERGENCY PLANT EVOLUTIONS

Facility: SONGS 2/3

Group I Emergency and Abnormal Plant Evolutions Target: 24% Actual: 24%

000003K104	Effects of power level and control position on delta flux	3.7
000005K304	Tech-Spec limits for inoperable rods	4.1
000011A103	Securing of reactor coolant pumps	4.0
000011A104	ESF actuation system in manual	4.4
000011K313	Hot-cold leg injection/recirculation	4.2
000015A122	RCP seal failure/malfunction	4.2
000015K102	Consequences of an RCPS failure	4.1
000024K301	When emergency boration is required	4.4
000026A204	Normal values and upper limits for temperature of the components cooled by the CCWS	2.9
000026K303	Guidance actions contained in EOP for Loss of CCW/nuclear service water	4.2
000029K301	Verifying a reactor trip: methods	4.5
000029A209	Occurrence of a main turbine/reactor trip	4.5
000040K101	Consequences of PTS	4.4
000040K105	Reactivity effects of a cooldown	4.4
000051A202	Conditions requiring reactor and/or turbine trip	4.1
000055A203	Ability to determine or interpret actions necessary to restore power	4.7
000055A206	Faults and lockout that must be cleared prior to re-energizing buses	4.1
000057A203	RPS alarm annunciators and trip indicators	3.9
000059K201	Radioactive-liquid monitors	2.8
000067K102	Fire Fighting	3.9
000068G010	Ability to perform without reference to procedures actions that require immediate operation of system components or controls	4.2
000069K203	Personnel access hatch and emergency access hatch	2.9
000074K103	Processes for removing decay heat from the core	4.9
000076G004	Knowledge of bases in technical specification for limiting conditions for operations and safety limits	3.7

Knowledge and Ability Record Form
EMERGENCY PLANT EVOLUTIONS

Facility: SONGS 2/3

Group II Emergency and Abnormal Plant Evolutions Target: 16% Actual: 16%

K/A	TOPIC	RATING SRO
000007G011	Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures	4.3
000007K106	Relationship of emergency feedwater flow to S/G and decay heat removal following reactor trip.	4.1
000008G007	Ability to explain and apply all systems limits and precautions	3.4
000009A238	Existence of head bubble	4.3
000009K321	Actions contained in EOP for small break LOCA/Leak	4.5
000025A101	RCS/RHRS cooldown rate	3.7
000025K101	Loss of RHRS during all modes of operation	4.3
000027A205	Pzr heater setpoints	3.3
000032K301	Startup termination on source-range loss	3.6
000033K302	Guidance contained in EOP for loss of intermediate-range instrumentation	3.9
000037K307	Actions contained in EOP for S/G tube leak	4.4
000038K306	Actions contained in EOP for RCS water inventory balance, steam tube rupture, and plant shutdown procedures	4.5
000054K303	Manual control of AFW flow control valves	4.1
000058K302	Actions contained in EOP for loss of dc power	4.2
000061K201	Detectors at each ARM system location	2.6
000065A206	When to trip reactor if instrument air pressure is decreasing	4.2

Knowledge and Ability Record Form
EMERGENCY PLANT EVOLUTIONS

Facility: SONGS 2/3

Group III Emergency and Abnormal Plant Evolutions Target: 3% Actual: 3%

K/A	TOPIC	RATING SRO
000028K302	Relationships between PZR pressure increase and reactor makeup/letdown imbalance	3.2
000036A104	Fuel handling equipment during an incident	3.7
000056K101	Principle of cooling by natural convection	4.2