February 21, 1996

Mr. M. S. Tuckman Senior Vice President Nuclear Generation Duke Power Company P.O. Box 1006 Charlotte, NC 28201

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - STEAM GENERATOR REPLACEMENT, CATAWBA NUCLEAR STATION, UNIT 1 (TAC NO. M90566)

Dear Mr. Tuckman:

On September 30, 1994, as supplemented on September 18, 1995, you submitted an application to change the Technical Specifications (TS) for the Catawba Nuclear Station, Units 1 and 2 in support of the replacement of the steam generators (SG) at Catawba Unit 1. The proposed TS changes would revise the low-low and high-high steam generator water level setpoints, the reactor coolant system volume, and the nominal average reactor coolant system temperature. It would also delete delete reference to repair methods, which are not applicable to the replacement steam generators and update topical report references in TS 6.9.

As a result of our review to date, we have identified a need for additional information as set forth in the Enclosure.

Sincerely,

Original signed by V. Nerses for:

Herbert N. Berkow, Director Project Directorate II-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosure: Request For Additional Information

cc w/encl: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

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As a result of our review to date, we have identified a need for additional information as set forth in the Enclosure.

Sincerely,

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Herbert N. Berkow, Director Project Directorate II-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

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cc w/encl: See next page

Duke Power Company

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Mr. William R. McCollum Site Vice President Catawba Nuclear Station Duke Power Company 4800 Concord Road York, South Carolina 29745

REQUEST FOR ADDITIONAL INFORMATION

STEAM GENERATOR REPLACEMENT

1. With respect to the proposed changes in steam generator (SG) low-low and high-high water level setpoints and the change in nominal average reactor coolant temperature, and the change in reactor coolant system volume, please identify which transients and accidents are affected from each TS change. Identify the most limiting transient or accident for the specified TS change. Provide supporting analysis results for the limiting case demonstrating how it meets the acceptance criteria with respect to system pressure and fuel performance (DNB, etc).

2. SR 4.4.5.4.b, TS page 3/4 4-16a

The standard TS reads as follows: "The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plug limit and all tubes containing through-wall cracks) required by Table 4.4-2." Duke Power did not include the phrase in the parentheses in its submittal. The staff believes that the phrase in the parentheses should be restored to the Catawba TS because the replaced steam generators are not exempt from this requirement.

3. BASES 3/4.4.5, TS page B 3/4 4-3a

Duke Power should delete the sentence that reads as follows: "Tubes experiencing outer diameter stress corrosion cracking within the thickness of the tube support plates are plugged or repaired by the criteria of 4.4.5.4.a.13." This statement is no longer applicable to Catawba Unit 1 after the replacement of the steam generators.

ENCLOSURE