

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2	PAGE (3) 1 OF 0 3
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TITLE (4)
Radiation Monitor Spike Causes Engineered Safety Features Actuations - Unknown Cause

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)													
0	1	2	0	8	8	8	8	8	0	0	1	0	0	0	2	1	5	8	8	0	5	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)

OPERATING MODE (9) 1	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 9.5	20.405(a)(1)(i)	50.38(e)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(e)
	20.405(a)(1)(ii)	50.38(e)(2)	<input type="checkbox"/>	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(vii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services	TELEPHONE NUMBER AREA CODE: 3 1 6 3 6 4 - 8 8 3 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	I	L	M	O	N	G	O	6	3
				N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen 4 1/2-line space typewritten lines) (16)

At approximately 1829 CST on January 20, 1988, a Fuel Building Ventilation Isolation Signal and a Control Room Ventilation Isolation Signal were received when the gas detector signal spiked high on Fuel Building Exhaust Radiation Monitor GG-RE-27. All required Engineered Safety Features equipment performed properly.

Following a review of the information available, interviews with personnel involved, and an event recreation, a definitive root cause could not be determined. No problems were found with monitor GG-RE-27 and it has experienced no further problems. Therefore, no corrective actions are planned at this time.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 8 8 - 0 0 1 - 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
						0 2 OF 0 3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

INTRODUCTION

At approximately 1829 CST on January 20, 1988, a Fuel Building Ventilation [VG] Isolation Signal (FBIS) and a Control Room Ventilation [VI] Isolation Signal (CRVIS) were received when the gas detector signal spiked high on Fuel Building Exhaust Radiation Monitor GG-RE-27[IL-MON]. All required Engineered Safety Features [JE](ESF) equipment performed properly.

DESCRIPTION OF EVENT

At approximately 1829 CST on January 20, 1988, with the unit in Mode 1, Power Operation, at approximately 95 percent power, a FBIS and CRVIS were received. No radiation above normal background readings was observed on redundant radiation monitor GG-RE-28. Monitor GG-RE-27 was then placed in bypass for troubleshooting at 1830 CST.

At the time that this event occurred, the Fuel Building Exhaust System was in operation to purge Freon-11 from the charcoal filters [VG-FLT] after completion of charcoal adsorber in-place leak testing. It was initially believed that the Freon-11 had caused the monitor to spike, but a recreation of the event on February 5, at approximately 1146 CST, disproved this theory.

After monitor GG-RE-27 was placed in bypass, several more spikes were observed. No spiking occurred on redundant monitor GG-RE-28. Monitor GG-RE-27 quit spiking and was returned to normal configuration at 1854 CST. The Fuel Building Exhaust System was restored and the FBIS was reset at approximately 2108 CST. The CRVIS was reset at approximately 2121 CST.

This event is being reported pursuant 10CFR 50.73(a)(2)(iv) as an automatic actuation of ESF equipment.

ROOT CAUSE

Following a review of the information available, interviews with Operations, Results Engineering, and Instrumentation and Controls personnel were conducted. Finally, an event recreation was performed. Based upon the results of this investigation, a definitive root cause could not be determined. No problems were found with monitor GG-RE-27 and has experienced no further problems. Therefore, no corrective actions are planned at this time.

ADDITIONAL INFORMATION

Radiation monitor GG-RE-27 is a particulate, iodine, and gas monitor, model 0356-1601-01 supplied by General Atomic Company.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

There was no damage to plant equipment or release of radioactivity as a result of this event. During this event, the redundant monitor was operable and performed the required system function. Therefore, at no time did conditions develop that posed a threat to the health or safety of the public.

A previous similar occurrence of unexplained spikes on another radiation monitor is reported in Licensee Event Report 85-001-00. Since this is considered to be an isolated event, the corrective actions taken in that report had no effect on this event.

WOLF CREEK

NUCLEAR OPERATING CORPORATION

Bart D. Withers
President and
Chief Executive Officer

February 15, 1988

WM 88-0043

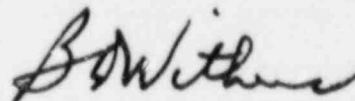
U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Subject: Docket No. 50-482: Licensee Event Report 88-001-00

Gentlemen:

The attached Licensee Event Report (LER) is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Features actuation.

Very truly yours,



Bart D. Withers
President and
Chief Executive Officer

BDW/llk

Attachment

cc: B. L. Bartlett (NRC), w/a
R. D. Martin (NRC), w/a
P. W. O'Connor (NRC), 2 w/a

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11