

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285  
 UNIT Fort Calhoun Station  
 DATE February 9, 1988  
 COMPLETED BY W. J. Blessie  
 TELEPHONE 402-536-4595

MONTH January 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	488.0	17	490.7
2	488.0	18	490.5
3	487.9	19	490.6
4	487.9	20	490.8
5	487.7	21	490.4
6	487.5	22	490.5
7	487.1	23	491.2
8	487.1	24	491.2
9	487.7	25	491.1
10	487.9	26	491.1
11	488.1	27	491.0
12	488.9	28	490.5
13	490.6	29	491.2
14	490.7	30	491.1
15	490.8	31	491.0
16	490.8		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

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OPERATING STATUS

1. Unit Name: Fort Calhoun Station Notes  
 2. Reporting Period: January 1988  
 3. Licensed Thermal Power (Mwt): 1500  
 4. Nameplate Rating (Gross Mwe): 502  
 5. Design Electrical Rating (Net MWe): 478  
 6. Maximum Dependable Capacity (Gross MWe): 502  
 7. Maximum Dependable Capacity (Net MWe): 478  
 8. If changes occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
N/A

9. Power Level to Which Restricted, If Any (Net MWe): N/A  
 10. Reasons for Restrictions, If Any: \_\_\_\_\_

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>125,810.0</u>
12. Number of Hours Reactor Was Critical	<u>744.0</u>	<u>744.0</u>	<u>97,583.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,309.5</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>96,662.8</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,109,355.4</u>	<u>1,109,355.4</u>	<u>125,656,980.6</u>
17. Gross Electrical Energy Generated (MWH)	<u>381,356.0</u>	<u>381,356.0</u>	<u>41,354,277.2</u>
18. Net Electrical Energy Generated (MWH)	<u>364,311.5</u>	<u>364,311.5</u>	<u>39,508,394.3</u>
19. Unit Service Factor	<u>100.0</u>	<u>100.0</u>	<u>76.8</u>
20. Unit Availability Factor	<u>100.0</u>	<u>100.0</u>	<u>76.8</u>
21. Unit Capacity Factor (Using MDC Net)	<u>102.4</u>	<u>102.4</u>	<u>67.8</u>
22. Unit Capacity Factor Using DER Net)	<u>102.4</u>	<u>102.4</u>	<u>65.9</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>3.1</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>N/A</u>			

25. If Shut Down at End of Report Period, Estimated Date of Startup: N/A  
 26. Units In Test Status (Prior to Commercial Operation): Forecast        Achieved

INITIAL CRITICALITY \_\_\_\_\_  
 INITIAL ELECTRICITY N/A \_\_\_\_\_  
 COMMERCIAL OPERATION \_\_\_\_\_

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January 1988

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No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns or power reductions during the month of January 1988.

<sup>1</sup>  
 F. Forced  
 S. Scheduled

<sup>2</sup>  
 Reason  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method  
 1 Manual  
 2 Manual Scram.  
 3 Automatic Scram.  
 4 Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

Refueling Information  
Fort Calhoun - Unit No. 1

Report for the month ending December 1987.

1. Scheduled date for next refueling shutdown. September 1988
2. Scheduled date for restart following refueling. December 1988
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
  - a. If answer is yes, what, in general, will these be?  

Incorporate cycle specific requirements resulting from reload safety analysis.
  - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. \_\_\_\_\_
  - c. If no such review has taken place, when is it scheduled? \_\_\_\_\_
4. Scheduled date(s) for submitting proposed licensing action and support information. July 1988
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:
 

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>393</u>	"
c) spent fuel pool storage capacity	<u>729</u>	"
d) planned spent fuel pool storage capacity	<u>May be increased via fuel pin consolidation</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by Kevin Helgeson Date January 26, 1988

OMAHA PUBLIC POWER DISTRICT  
Fort Calhoun Station Unit No. 1

January 1988  
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% power throughout January 1988. Construction continued on the training facility, warehouse and maintenance shop.

NRC inspections were performed on Security, Design Changes, Solid Radioactive Waste and Quality Assurance.

A team was formed to conduct an assessment of 20 key functional areas relating to Fort Calhoun Station. The team is to identify the improvements necessary to achieve the desired standards of excellence in operations at the Fort Calhoun Station.

Presentations were made to 39 small groups on commitment to excellence.

No safety valves or PCRV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure

Description

SP-FAUD-1

Fuel Assembly Uplift Condition Detection.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.



F. CHANGES IN PLANT OPERATING STAFF

During January, Mr. Delvin R. Trausch was promoted to Supervisor-Operations.

G. TRAINING

During January, Station Training completed training and qualification of two shift technical advisors, began a three month technical staff class for six individuals, continued initial auxiliary operator-nuclear and equipment operator-nuclear training, continued licensed and non-licensed training, and completed training of two initial hot license candidates and one upgrade candidate in preparation for the examinations during the week of February 8, 1988. In addition, preparations are being made for the requalification examinations using the process described in the pilot program.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None.

II. MAINTENANCE (Significant Safety Related)

None

*W. Gary Gates*

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Manager-Fort Calhoun Station