



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

October 14, 1994

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT  
HOPE CREEK GENERATION STATION UNIT 1  
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for September are being forwarded to you with the summary of changes, tests, and experiments that were implemented during September 1994 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

*MIL*  
R. J. Hovey  
General Manager -  
Hope Creek Operations

*Duk. 10/14/94*  
DR:WS:JC  
Attachments

C Distribution

190029

*10/24*  
*11*

The Energy People

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PDR ADOCK 05000354  
R PDR

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OPERATING DATA REPORT

DOCKET NO. 50-354  
 UNIT Hope Creek  
 DATE 10/11/94  
 COMPLETED BY D. W. Lyons  
 TELEPHONE (609) 339-3517

OPERATING STATUS

1. Reporting Period September 1994 Gross Hours in Report Period 720

2. Currently Authorized Power Level (Mwt) 3293  
 Max. Depend. Capacity (MWe-Net) 1031  
 Design Electrical Rating (MWe-Net) 1067

3. Power Level to which restricted (if any) (MWe-Net) None

4. Reasons for restriction (if any)

	<u>This Month</u>	<u>Yr To Date</u>	<u>Cumulative</u>
5. No. of hours reactor was critical	<u>719.6</u>	<u>5109.9</u>	<u>57932.9</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours generator on line	<u>713.0</u>	<u>4999.9</u>	<u>57032.4</u>
8. Unit reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross thermal energy generated (MWH)	<u>2312673</u>	<u>16015088</u>	<u>181978458</u>
10. Gross electrical energy generated (MWH)	<u>766071</u>	<u>5298761</u>	<u>60262715</u>
11. Net electrical energy generated (MWH)	<u>733441</u>	<u>5055097</u>	<u>57582781</u>
12. Reactor service factor	<u>99.9</u>	<u>78.0</u>	<u>84.9</u>
13. Reactor availability factor	<u>99.9</u>	<u>78.0</u>	<u>84.9</u>
14. Unit service factor	<u>99.0</u>	<u>76.3</u>	<u>83.6</u>
15. Unit availability factor	<u>99.0</u>	<u>76.3</u>	<u>83.6</u>
16. Unit capacity factor (using MDC)	<u>98.8</u>	<u>74.8</u>	<u>81.9</u>
17. Unit capacity factor (Using Design MWe)	<u>95.5</u>	<u>72.3</u>	<u>79.1</u>
18. Unit forced outage rate	<u>1.0</u>	<u>5.2</u>	<u>4.5</u>
19. Shutdowns scheduled over next 6 months (type, date, & duration):	<u>None</u>		
20. If shutdown at end of report period, estimated date of start-up:	<u>N/A</u>		

OPERATING DATA REPORT  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354  
UNIT Hope Creek  
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COMPLETED BY D.W. Lyons  
TELEPHONE (609) 339-3517

MONTH September 1994

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
1	9/1	F	7 (TOTAL OF 45)	A	4 CONTIN- UATION	UNIT SYNCHRONIZED ON 09/01/94 AFTER COMPLETION OF REPAIRS.
2	9/3	S	0	B	5 POWER REDUCTION ONLY	POWER REDUCED MORE THAN 20% FOR ROD ADJUSTMENT, & CIV SURVEILLANCE TESTS.

AVERAGE DAILY UNIT POWER LEVEL

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UNIT Hope Creek  
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MONTH September 1994

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. 312  
2. 1012  
3. 978  
4. 1060  
5. 1056  
6. 1067  
7. 1030  
8. 1050  
9. 1039  
10. 1047  
11. 1053  
12. 1041  
13. 1057  
14. 1045  
15. 1042  
16. 1040

17. 1035  
18. 1037  
19. 1057  
20. 1051  
21. 1049  
22. 1039  
23. 1034  
24. 1044  
25. 1032  
26. 1041  
27. 1045  
28. 1032  
29. 1064  
30. 1067  
31. n/a



HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

September 1994

Hope Creek entered the month of September shut down due to a plant trip on August 30th caused by a loss of stator water cooling. The plant was restarted on September 1, 1994. After attaining 100% power on September 2, 1994, power was reduced greater than 20% on September 3, 1994 to allow for rod adjustment and Combined Intermediate Valve testing. The plant was operated at 100% for the remainder of the month. As of September 30, 1994 the plant has been on line for 30 consecutive days.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS

FOR THE HOPE CREEK GENERATING STATION

September 1994

The following items have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.



## Temporary Modification Summary of Safety Evaluation

T-Mod 94-024: This Temporary Modification installed an electrical jumper across the #2 Feedwater Heater Hi-Hi Level trip switches and installed a temporary keep fill line to the low side of the level transmitters. This modification is performed due to spurious indications during power ascension and is removed at approximately 40 % Reactor Power. This T-Mod does not increase the probability or the consequences of an accident listed in Table 15.0-2 of the UFSAR since the worst case would be for water induction into the turbine resulting in a turbine trip.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

T-Mod 94-026: This temporary modification is a change to the normal line-up of the test line for the MSIV Seal System return to PSIG. It includes the addition of a drain hose to the drain leg of the test return line valve. The system is designed to utilize this point as a drain. The only change is from the normal system valve line-up and the addition of the drain hose.

This modification changes the plant as described in the UFSAR in that the P&ID describes the normal alignment of the system without a drain hose as shown in the UFSAR.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

T-Mod 94-027: This Temporary Modification raises the reset point for the flow switch in the SACS system. The flow switch is causing spurious trips when the "C" pump is switched from auto to manual when the "B" loop of SACS is supplying TACS. The trips are being caused by momentary pressure oscillations in the SACS system which, when combined with a suspected leakage past the 1EG-HV-2522A & C valves, is causing a pressure spike, which resets the flow switch and then when the spike disappears, causes the flow switch to trip.

This T-Mod will move the reset point high enough to prevent the switch from improperly resetting and will allow the system to function properly until the suspected valve leakage can be corrected. This modification has sufficient margin to the normal flow level and will not impact normal system operation.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

T-Mod 94-028: This Temporary Modification creates an automatic control loop for operating the Auxiliary Boiler Steam Dump Valve. The purpose for this modification is to limit the amount of steam exhausted to the atmosphere as this will reduce fuel consumption and increase boiler efficiency. The present configuration does not automatically manipulate the dump valve open or closed based on load demand since the air signal is input manually in order to set valve position. As a result, the valve is opened continuously exhausting steam to the atmosphere.

This modification attempts to automate the operation of the dump valve by utilizing steam header pressure as a process input signal to control the dump valve open or closed. Automatic operation of the dump valve is achieved by placing the Selector Switch in VENT Position to control the dump valve position utilizing the Auxiliary Steam Header Pressure.

The auxiliary steam system has no safety related function. The system is designed so that a failure of the system or component does not compromise any safety related system or component or prevent a safe reactor shutdown.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

T-Mod 94-029: This Temporary Modification installed an electrical bypass jumper on "D" Service Water Traveling Water Screen (TWS) Spray Wash Booster Pump flow switch. This jumper will simulate a flow signal to allow the "D" TWS to operate until a replacement flow switch can be procured and installed. When the TWS is in AUTO, the spray pump will start upon the start of it's associated Service Water Pump. Should there be a failure of this T-Mod the spray wash booster pump will trip which will cause the TWS to stop. Debris will enter the self-cleaning strainer which will remove this debris before reaching the Station Service Water System (SSWS). An alarm will also be generated by CRIDS in the main control room for differential water level on the TWS speed status.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

H-1-ZZ-NSE-0811: This evaluation addresses on a generic basis a subset of proposed temporary modifications that may be implemented for the purpose of bypassing control room overhead annunciator input signals which have alarmed and cannot be immediately corrected by maintenance. The alarm represents a nuisance distraction to control room operators, may mask other contributions to the alarm window and provide no useful information to the operator since the initiating condition has been recognized and appropriate action to correct the condition would be initiated in accordance with applicable alarm response procedures and the work control program. Additionally, Operations procedures provide for marking of alarm windows which have inoperable or modified inputs.

This subset of T-Mods will be controlled by the Temporary Modification Procedure which requires Licensed Operator approval, independent verification, System Engineering review, and appropriate periodic management review to ensure that the T-Mods are adequately administered and controlled.

Therefore, this generic Safety Evaluation for this subset of Temporary Modifications does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

#### Procedure Summary of Safety Evaluation

NC.NA-AP.ZZ-0047(Z) Rev 1: This procedure titled "Nuclear Support Facility Management Program", has been deleted and replaced by a Nuclear Department Business Procedure NC.NA-BP.ZZ-003(Z) to cover this information. The operational Quality Assurance program does not apply to this procedure. Although this procedure is briefly described in the UFSAR (Section 13.5.1 Hope Creek Plant Procedures) it performs no function relative to the safe operation of Hope Creek.

Therefore, this Procedure revision does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

Other Summary of Safety Evaluation

UFSAR CN #94-36: This UFSAR Change Notice addresses the Structural Steel Fireproofing Tables (9A-103 and 9A-104) which are being deleted in the UFSAR in favor of direct reference to the Structural Steel Fireproofing Drawings (Architectural Drawings A-0241 through A-248). The information contained within the Tables is redundant to the information contained on these Drawings. Elimination of these tables will simplify the revision process, and reduce to one source for room fireproofing status. Text changes also included are mainly editorial in nature to make the information contained in the fireproofing program consistent with the information described on the design drawing. There are no parameters or systems affected by this change.

Therefore, this UFSAR Change Notice does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.