Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

February 10, 1988

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period January 1, 1988 through January 31, 1988.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/SJS/sjc

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JANUARY 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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DOCUMENT ID 0043r/0031r

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761, extension 705.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

January 1-31

January 1, 0001 hours	Reactor critical, generator on-line at 960 MWe.
January 8, 2200 hours	Load drop to 635 MWe for MSIV Surveillances and rod set.
January 9, 1600 hours	Commence ramping to 1090 MWe over a two day period.
January 16, 2300 hours	Load drop to 870 MWe at request of Load Dispatcher.
January 17, 0600 hours	Commence ramping to 1020 MWe.
January 29, 2330 hours	Load drop to 569 MWe for rod adjustments.
January 30, 0300 hours	Commence ramping to 842 MWe.
January 31, 0300 hours	Ramping to 1030 MWe.
January 31, 2400 hours	Reactor critical, generator on-line at 980 MWe, continuing ramp to 1030 MWe.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility License or Technical Specification.

   Amendment number: 52

   The facility license was amended to allow transfer of Unit 1 and 2 fuel to either fuel pool.

2. Changes to procedures which are described in the Safety Analysis Report.

   There were no changes to procedures which are described in the Safety Analysis report.

3. Tests and Experiments not described in the Safety Analysis Report.

   There were no tests or experiments conducted during this reporting period that are not described in the Safety Analysis Report.

4. Corrective Maintenance of Safety Related Equipment.

   The following table (Table 1) presents a Summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

5. Completed Safety Related Modifications.

   The following Table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L75461	Supp. Pool Temp Recorder, 1TR-CM037	Bad card	False upscale alarms	Replaced card
L74698	"1D" APRM 1C51-R901D	Bad wire	APRM failed downscale	Repaired wire
L59799	250 VDC Charger 1DC-03E	Bad amp board	Improper load supply	Replaced amp board
L74032	"1A" Diesel air compressor, 1DG-08CB	Failed gasket	Air leaking from compressor	Replaced gasket
L73611	"1B" SBLC Pump, 1C41-C001B	Sight Glass oil leak	Oil leaking from crankcase	Replaced sight glass
L75206	"1B" SBLC Relief valve flange 1C41-F029B	Valve bellows failed	Flange leaking	Replaced bellows
L74855	S.P. Air temp/press. Hi alarm	Broken pin connector	False alarm, would not reset.	Replaced pin connector
L74891	Scram pilot solenoids 1C11-D001-118	Solenoids failed	Control rod went full in during 1/2 scram testing	Replaced Solenoids 1C11-D001.
L74893	Div. II SP/DW Temp. Recorder 1TR-CM038	Chart motor failed.	Chart not advancing.	Replaced slow speed chart motor.
L74919	"F" APRM 1C51-K605CS	Dirty edge connector	No downscale light.	Cleaned edge connector.

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER:	A brief synopsis of incorporated modification objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT ONE

There were no safety related modificatons completed for Unit One during the reporting period, January 1, to January 31, 1988.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, January 1, 1988 through January 31, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence

There were no Licensee Event Reports logged during the report period January 1, through January 31, 1988.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE February 10, 1988
COMPLETED BY S. J. Samolinski
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: January, 1988 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 1025
4. REASONS FOR RESTRICTION (IF ANY): Administrative

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	744.0	20788.5
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	1642.0
7.	HOURS GENERATOR ON LINE	744.0	744.0	20174.8
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	2233488	2233488	54067786
10.	GROSS ELEC. ENERGY GENERATED (MWH)	755786	755786	17723045
11.	NET ELEC. ENERGY GENERATED (MWH)	730250	730250	16828891
12.	REACTOR SERVICE FACTOR	100.0%	100.0%	58.0%
13.	REACTOR AVAILABILITY FACTOR	100.0%	100.0%	62.6%
14.	UNIT SERVICE FACTOR	100.0%	100.0%	56.3%
15.	UNIT AVAILABILITY FACTOR	100.0%	100.0%	56.3%
16.	UNIT CAPACITY FACTOR (USING MDC)	94.7%	94.7%	45.3%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	91.0%	91.0%	43.6
18.	UNIT FORCED OUTAGE RATE	0.0%	0.0%	13.8%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
The Unit One second refuel outage is scheduled to begin March 13, 1988 and will last 15 weeks.

20. IF SHUT DOWN IS AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: February 10, 1988
COMPLETED BY: S. J. Samolinski
TELEPHONE: (815) 357-6761
MONTH: JANUARY, 1987

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	913	17.	936
2.	907	18.	952
3	970	19.	1028
4.	1034	20.	1037
5.	1029	21.	1026
6.	1032	22.	1036
7.	1019	23.	1033
8.	973	24.	1032
9.	723	25.	1039
10.	826	26.	1041
11.	986	27.	1040
12.	1045	28.	1020
13.	1042	29.	1030
14.	1043	30.	726
15.	1039	31.	871
16.	1000		

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE February 10, 1988
COMPLETED BY S. Samolinski
TELEPHONE (815)357-6761

REPORT MONTH JANUARY, 1988

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
1	1/8/88	S	0.0	B	5	MSIV Surveillances and rod set.
2	1/30/88	S	0.0	B	5	Rod set.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-2-88	HPCS CY Suction Valve 1E22-F001	Replace limiter plate and leads 14 and 14c
1-5-88	Petter diesel air compressor 1E22-C302B	Prevent operation
1-10-88	1E51-C004	Inspect motor and brushes on pump
1-11-88	1E51-C005	Inspect motor and brushes on pump
1-18-88	1E12-F003A	Surveillance LES-EQ-112
1-53-88	1B D/G Air Compressor	Replace head gasket
0-2-88	0 D/G	Repair air start solenoid
0-21-88	0 D/G	Rebuild air start solenoids
0-23-88	0 D/G	BOP calibrations

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

A new contract was awarded to our present vendor. The new contract will cover a two year period. More information will be provided in February's report.

5. Indications of Failed Fuel Elements

There were no indications of Failed Fuel Elements during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JANUARY, 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0031r

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761 extension 705.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

JANUARY 1-31

January 1, 0001 hrs.	Reactor critical, generator on-line at 840 MWe.
January 3, 1750 hrs.	Load drop to 620 MWe to repair pipe on "B" TDRFP.
January 4, 0400 hrs.	Commence ramping to 915 MWe.
January 6, 0900 hrs.	Ramping to 1115 MWe.
January 13, 1850 hrs.	Load drop to 830 MWe for heater bay work and valve Surveillance.
January 14, 0500 hrs.	Commence ramping to 1110 MWe.
January 16, 2300 hrs.	Load drop to 850 MWe at request of Load Dispatcher.
January 17, 0600 hrs.	Commence ramping to 1030 MWe.
January 19, 0000 hrs.	Load drop to 850 MWe at request of L.D.
January 19, 0600 hrs.	Commence ramping to 1116 MWe.
January 20, 0000 hrs.	Load drop to 850 MWe at request of L.D.
January 20, 0500 hrs.	Commence ramping to 1110 MWe.
January 22, 2200 hrs.	Load drop to 840 MWe for Surveillances and reheater work.
January 27, 2300 hrs.	Load drop to 850 MWe at request of L.D.
January 28, 0300 hrs.	Commence ramping to 1110 MWe
January 30, 2300 hrs.	Load drop to 700 MWe for rod adjustments and scram times.
January 31, 0700 hrs.	Commence ramping to 938 MWe.
January 31, 2300 hrs.	Load drop to 672 MWe for rod adjustment.
January 31, 2400 hrs.	Reactor Critical, Generator on-line and holding at 670 MWe.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility license or Technical Specification.

   Amendment Number: 34
   The facility license was amended to allow transfer of Unit 1 and 2 fuel to either fuel pool.

2. Changes to procedures which are described in the Safety Amalysis Report.

   There were no changes to procedures which are described in the Safety Analysis Report.

3. Tests and Experiments not described in the Safety Analysis Report.

   There were no Tests or Experiments conducted during this reporting period that are not described in the Safety Analysis Report.

4. Corrective Maintenance of Safety Related Equipment.

   The following table (Table 1) presents a summary of Safety-Related Maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

5. Completed Safety Related Modifications.

   The following table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L73645	RCIC steam line pressure switch 2E31-N007AA	Failed diaphragm	Improper switch operation	Replaced switch
L73669	RCIC Steam line isolation switch 2E31-N007AB	Failed diaphragm	Imrpoper switch operaton	Replaced switch
L72610	Battery cells 2E22-S001	Low specific gravity	Low battery charge	Battery cells put on individual change.
L75231	LPCS water leg pump disc check valve 2E21-F033	Failed valve seat	Improper valve seal	Rebuilt valve with new spring and gasket
L74742	"A" RBM 2C51-K605GU	Bad Relay	Erroneous rod blocks	Replaced K11 relay.
L74778	"A" RBM 2C51-K605GU	Failed Relays	Improper response	Replaced relays.
L74739	LPCS pump cooler upstream stop 2DG035	Inadequate lubrication	Valve sticking	Manually exercised valve and lubed valve stem.
L74696	"A" APRM flow bias	Worn wiper assembly	Improper indication	Replaced wiper assembly
L73548	"B" APRM recorder 2C51-R603B	Loose clutch	Pen not advancing	Tightened clutch
L74253	"B" SBLC Pump motor 2C41-C001B	Cracked overload device	Exposed contacts	Replaced overload device

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L75303	RHR Area high temp Riley module 2E31-N608C	Failed module	Improper area temp. indication	Replaced Riley module
L71954	HPCS diesel air receiver tank 2E22-T302	Improper seal	Leak in air line	Removed line, cleaned, applied new sealant
L74733	S/P Temp. recorder 2TR-CM037	Bad pinion gear	Improper recorder operation	Replaced pinion gear
L75179	S/P Temp. recorder 2TR-CM037	Bad sync cord and signal level cord	Recorder not advancing properly	Replaced sync cord and cleaned signal cord
L75458	A/C APRM recorder 2C51-R603A	Loose clutch and bad wiper arm assembly	Improper power indication	Tightened clutch and replaced wiper assembly

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER:	A brief Synopsis of Incorporated Modification Objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT TWO

M-1-2-86-091: Snubber reduction on subsystem 2HG70 per the snubber reduction program.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, January 1, through January 31, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence

There were no Licensee Event Reports logged during the reporting period January 1 through January 31, 1988.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE February 10, 1988
COMPLETED BY Steven J. Samolinski
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: January, 1988 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY): None

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	744.0	17528.8
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	29.83
7.	HOURS GENERATOR ON LINE	744.0	744.00	17215.1
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	2247120	2247120	50042176
10.	GROSS ELEC. ENERGY GENERATED (MWH)	753086	753086	16561297
11.	NET ELEC. ENERGY GENERATED (MWH)	727007	727007	15809620
12.	REACTOR SERVICE FACTOR	100.0%	100.0%	60.8%
13.	REACTOR AVAILABILITY FACTOR	100.0%	100.0%	60.9%
14.	UNIT SERVICE FACTOR	100.0%	100.0%	59.7%
15.	UNIT AVAILABILITY FACTOR	100.0%	100.0%	59.7%
16.	UNIT CAPACITY FACTOR (USING MDC)	90.7%	94.3%	52.9%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	90.7%	90.7%	50.9%
18.	UNIT FORCED OUTAGE RATE	0.0%	0.0	19.9%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): No shutdowns are scheduled during the next six months.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP. N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: February 10, 1988
COMPLETED BY: Steven J. Samolinski
TELEPHONE: (815) 357-6761
MONTH: DECEMBER 1987

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	809	17.	940
2.	816	18.	1023
3	782	19.	986
4.	804	20.	996
5.	877	21.	1051
6.	942	22.	1015
7.	1067	23.	872
8.	1068	24.	942
9.	1066	25.	1011
10.	1068	26.	1046
11.	1035	27.	1063
12.	1034	28.	985
13.	1011	29.	1036
14.	985	30.	1053
15.	1040	31.	801
16.	1068		

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE February 10, 1988
COMPLETED BY S. Samolinski
TELEPHONE (815)357-6761

REPORT MONTH January, 1988

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
1	1/31/88	S	0.0	B	5	Load drop for rod adjustments.

DOCUMENT ID 0936r/0031r

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety/Relief Valve operated for Unit #2 during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-1-88	2B D/G Strainer Backwash 2E22-F319	During backwash valve goes full open
2-7-88	2B D/G Cooling Water Strainer	Reset backwash vlv. limits
2-8-88	HPCS D/G	Misc. D/G work
2-9-88	HPCS D/G	Calibrate VAR meter
2-10-88	HPCS D/G	Lubrication
2-12-88	HPCS Water leg pump	Lubrication
2-13-88	HPCS Pump	Change oil
2-19-88	LPCS motor cooling valve	Troubleshooting
2-21-88	C RHR Service Water pump	Lubrication
2-22-88	D RHR Service Water pump	Lubrication
2-23-88	B/C RHR Water Leg Pump	Lubrication
2-24-88	2B RHR Pump	Change oil
2-25-88	2C RHR Pump	Change oil
2-26-88	C RHR High point vent 2E31-D311C	Repair sightglass leak
2-28-88	C/D RHR pump strainer	Change oil
2-37-88	RHR Service Water pumps A/B	Change oil
2-38-88	2A Service Water Strainer	Change oil
2-39-88	2A RHR	Lubrication
2-40-88	LPCS	Lubrication
2-41-88	RHR 2A service water pump	Lubrication
2-42-88	RHR 2B service water pump	Lubrication
2-43-88	Division 1 LPCS/RHR water leg pump	Lubrication
2-53-88	LPCS water leg pump 2E21-C002	Cleaning
2-54-88	RHR shutdown cooling valves	Repair 2E31-N068C

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

A new contract was awarded to our present vendor. The new contract will cover a two year period. More information will be provided in February's report.

5. Indications of Failed Fuel Elements.

Off Gas Levels indicate one pinhole fuel element failure in the reactor. This does not represent a change from the previous reporting period.

There were no indications of Failed Fuel Elements during this reporting period.