

DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 07/05/97
COMPLETED BY: F. Todd
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AVERAGE DAILY UNIT POWER LEVEL

MONTH MAY 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1046</u>	17	<u>1055</u>
2	<u>1052</u>	18	<u>1054</u>
3	<u>1049</u>	19	<u>1038</u>
4	<u>1043</u>	20	<u>1047</u>
5	<u>1057</u>	21	<u>1052</u>
6	<u>1056</u>	22	<u>1055</u>
7	<u>1058</u>	23	<u>1050</u>
8	<u>1054</u>	24	<u>1049</u>
9	<u>1058</u>	25	<u>1042</u>
10	<u>1053</u>	26	<u>1040</u>
11	<u>1032</u>	27	<u>945</u>
12	<u>1052</u>	28	<u>1044</u>
13	<u>1054</u>	29	<u>1050</u>
14	<u>1057</u>	30	<u>1047</u>
15	<u>1045</u>	31	<u>859</u>
16	<u>1051</u>		

DOCKET NO.: 50-354
UNIT: Hope Creek
D.F.E: 07/05/97
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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

MONTH JUNE 1997

The following items completed during May 1997 have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

Radwaste Asphalt Drum Level Instrumentation Installation, 4EC-03161, Package 1, Revision 0. This design change modified the solid radwaste asphalt drum filling level monitoring and control loops by: 1) replacing the ultrasonic level transmitters with microprocessor based, parameter programmable, ultrasonic devices; 2) changing the scheme used to select the lead and standby fill stations; and 3) removing the hi hi trip of the extruder. Since the fill operation is a slow procedure and is monitored by closed circuit video, the operator will be able to terminate the filling operation without reliance on the hi hi trip. A hi alarm will alert the operator that the drum is close to full. UFSAR section 11.4.2.4, which described the level indication and control loops, was changed. The asphalt drum system, its operation, or its failure do not contribute to or initiate any accident scenario postulated in the UFSAR. The asphalt drum system is not used to mitigate or preclude any accident described in the UFSAR. The

asphalt drum system does not communicate, either physically or electrically, with any system important to safety.

Therefore, this design change does not involve an Unreviewed Safety Question.

Replacement of Air-Operated Valves in the Safety Auxiliaries Cooling System (SACS), 4EC-03612, Package 13, Revision 0. The Residual Heat Removal (RHR) pump room cooler water supply valves in the SACS were replaced. The previous valves were carbon steel air actuated flexible-wedge gate valves. The replacement valves are stainless steel air actuated ball valves which are more suitable to the application. UFSAR Figure 9.2-4 was changed to show the type and material of the replacement valves/actuators. UFSAR Table 3.9-18 was changed to show the valve/actuator replacement type. This replacement does not change the function of the valves or the function of the affected systems. The change does not alter the ability of the SACS, the Equipment Area Cooling System (EACS), or the RHR pump room unit coolers to perform their intended safety functions. The affected systems will function in accordance with the original design and licensing basis of the plant.

Therefore, this design change does involve an Unreviewed Safety Question.

Replacement of Air-Operated Valves in the Safety Auxiliary Cooling System (SACS), 4EC-03612, Package 18, Revision 0. The High Pressure Coolant Injection (HPCI) pump room cooler water supply valves in the SACS were replaced. The previous valves were carbon steel air actuated flexible-wedge gate valves. The replacement valves are stainless steel air actuated ball valves which are more suitable to the application. UFSAR Figure 9.2-4 was changed to show the type and material of the replacement valves and actuators. UFSAR Table 3.9-18 was changed to show the valve/actuator replacement type. This replacement does not change the function of the valves or the function of the affected systems. The change does not alter the ability of the SACS, the Equipment Area Cooling System (EACS), or the HPCI pump room unit coolers to perform their intended safety function. The affected systems will function in accordance with the original design and licensing basis of the plant.

Therefore, this design change does not involve an Unreviewed Safety Question.

Replacement of Air-Operated Valves in the Safety Auxiliaries Cooling System (SACS), 4EC-03612, Package 22, Revision 0. The Filtration, Recirculation, Ventilation System (FRVS)

Recirculation System cooling coil water supply valves in the SACS were replaced. The previous valves were carbon steel air actuated flexible-wedge gate valves. The replacement valves are stainless steel air actuated ball valves which are more suitable to the application. UFSAR Figure 9.2-4 was changed to show the type and material of the replacement valves and actuators. UFSAR Table 3.9-18 was changed to show the valve/actuator replacement type. This replacement does not change the function of the valves or the function of the affected systems. The change does not alter the ability of the SACS or the FRVS Recirculation System to perform their intended safety functions. The affected systems will function in accordance with the original design and licensing basis of the plant.

Therefore, this design change does not involve an Unreviewed Safety Question.

Replacement of Air-Operated Valves in the Safety Auxiliaries Cooling System (SACS), 4EC-03612, Package 29, Revision 0. The Core Spray (CS) pump room cooler water supply valves in the SACS were replaced. The previous valves were carbon steel air actuated flexible-wedge gate valves. The replacement valves are stainless steel air actuated ball valves which are more suitable to the application. UFSAR Figure 9. 4 was changed to show the type and material of the replacement valves and actuators. UFSAR Table 3.9-18 was changed to show the valve/actuator replacement type. This replacement does not change the function of the valves or the function of the affected systems. The change does not alter the ability of the SACS, the Equipment Area Cooling System (EACS), or the CS pump room unit coolers to perform their intended safety functions. The affected systems will function in accordance with the original design and licensing basis of the plant.

Therefore, this design change does not involve an Unreviewed Safety Question.

Station Service Water System (SSWS) Pumps Pit Level Transmitters Removal/Abandonment, 4HE-00265, Package 1, Revision 0. This design change removed the SSWS pumps suction level instrumentation loops (1EALIT-2241A through 1EALIT-2241D and their associated instruments). These were non-safety related loops that were designed to monitor the water levels in the SSWS pump bays for recording/indication in the Main Control Room. These indications were not used by the operators nor were they required for the operation of the SSWS pumps. The operating procedures did not rely upon these instrumentation loops for any safety-related function or operation. Removal of these instruments did not affect the operation of the SSWS nor did it

affect the design basis of the system. This design change did not involve any unique tests or experiments.

Therefore, this design change does not involve an Unreviewed Safety Question.

Remove Flow Switch 1EPFS-2225C, 4HE-00356, Package 3, Revision 0. Previously implemented and reported design change 4EC-03453, Package 1 (erroneously reported in the December 1994 Monthly Operating Report as design change 4EC-5343, Package 1) modified the station service water system (SSWS) traveling screen logic and disconnected the cables of the service water screen and backwash flow switch 1EPFS-2225C, but left the flow switch in place. Design change 4HE-00356 removed the flow switch, 1EPFS-2225C, and installed gaskets and a blind flange in place of the flow switch. Stress and seismic evaluations have been performed to demonstrate the adequacy of the change. The replacement of the flow switch with a blind flange does not affect the amount of flow being provided to the Reactor Auxiliary Cooling System (RACS) and Safety Auxiliary Cooling System (SACS) heat exchangers during plant conditions, transients, and accident conditions. The change does not: 1) change, degrade, or prevent actions described or assumed in any accident described in the UFSAR, 2) alter any assumptions previously made in evaluating radiological consequences of any accident described in the UFSAR, 3) affect the mitigation of the radiological consequences of any accident described in the UFSAR, 4) affect a fission product barrier, or 5) change the composition or inventory of radioactivity releases. This change does not adversely affect the operability of the Station Service Water System (SSWS).

Therefore, this design change does not involve an Unreviewed Safety Question.

UFSAR Change Notices Summary of Safety Evaluations

UFSAR Change Notice H97-022, UFSAR Sections 13 & 17, Transfer of Quality Verification (QV) from Manager - Corrective Action & Quality Services (CA&QS) to the Manager - Quality Assessments (QA). This UFSAR change reflects a realignment of the QV functions from the Manager - CA&QS to the Manager - QA. The realignment of responsibility for the QV functions to the Manager - QA has no direct affect on any systems, structures, or components as described in the Salem and Hope Creek UFSARs. This change will not reduce the level of management control and authority, responsibilities, training or qualifications of personnel, or any QA program commitments. The Quality Assurance program, as described in the Hope Creek and Salem UFSAR Section 17.2, will continue to meet the criteria of 10CFR50 Appendix B.

Therefore, this UFSAR change does not involve an Unreviewed Safety Question.

UFSAR Change Notice H97-026, UFSAR Sections 13.1 & 17.2, Transfer of Project Managers and Functions from Nuclear Engineering to Nuclear Business Support. This change transferred the responsibility for Project Management from Nuclear Engineering to Nuclear Business Support in UFSAR Sections 13.1 and 17.2. The change realigned organizational relationships to affect a more positive control of Nuclear Business Unit (NBU) activities. The change had no effect on any systems, structures, or components described in the UFSAR. The change did not result in any reduction in the qualifications and training requirements of personnel, technical support for plant operations, management control and authority, administrative program commitments or quality assurance program. This change meets the intent of the NUREG 0800. The revised organization will improve alignment of responsibilities and management focus on activities related to plant safety.

Therefore, this UFSAR change does not involve an Unreviewed Safety Question.

Procedures Summary of Safety Evaluations

NC.NA-AP.ZZ-0035(Q), Revision 7, Nuclear Licensing and Reporting. This procedure revision deletes the attachment that detailed the description of the station reporting requirements and served as a reference for the station staff to indicate which reports are required by federal, state, and local agencies. The text of the procedure continues to provide sufficient guidelines for the preparation, review, and submittal of correspondence to the NRC. This is an administrative change only.

Therefore, this procedure change does not involve an Unreviewed Safety Question.

Temporary Modifications Summary of Safety Evaluations

Deficiency Reports Summary of Safety Evaluations

Other Summary of Safety Evaluation

There were no changes in these categories implemented during May 1997.