Omaha Public Power District
444 South 16th Street Mall
Omaha NE 68102-2247

July 15, 1997
LIC-97-0115

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

Reference: Docket No. 50-285

SUBJECT: June 1997 Monthly Operating Report (MOR)

Enclosed please find the June MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification [illegible].

If you should have any questions, please contact me.

Sincerely,

S. K. Gambhir
Division Manager -
Engineering & Operations Support

SKG/mle

Enclosures

c: Winston & Strawn
E. W. Merschoff, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. J. Simon, Westinghouse
INPO Records Center

180035

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT NAME FORT CALHOUN STATION
DATE JULY 8, 1997
COMPLETED BY M. L. EDWARDS
TELEPHONE (402) 533-6929

REPORT PERIOD: June 1997

1. Design Electrical Rating (Mwe-Net): 478

2. Maximum Dependable Capacity (Mwe-Net): 478

OPERATING STATUS

	THIS MONTH	YR-TO-DATE	CUMULATIVE
3. Number of Hours Reactor was Critical:	720.0	4343.0	208321.0
4. Number of Hours Generator was On-line:	720.0	3802.4	162670.2
5. Unit Reserve Shutdown Hours:	0.0	0.0	0.0
6. Net Electrical Energy Generated (MWH):	344878.4	1797154.1	68783275.4

ATTACHMENT II
UNIT SHUTDOWNS

REPORT MONTH June 1997

DOCKET NO. 50-285
UNIT NAME Fort Calhoun
DATE July 8, 1997
COMPLETED BY M. L. Edwards
TELEPHONE (402) 533-6929

No.	Date	Type F: Forced S: Scheduled	Duration (Hours)	Reason[1]	Method of Shutting Down Reactor[2]	Cause & Corrective Action to Prevent Recurrence
None						

(1)
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training/License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

(2)
Method:
1-Manual
2-Manual Trip/Scram
3-Automatic Trip/Scram
4-Continuation
5-Other (Explain)

OPERATIONS SUMMARY:

Fort Calhoun Station (FCS) operated at a nominal 100% power level until 1958 hours on June 18, 1997 when reactor power was reduced to 95% to facilitate turbine control valve testing. Turbine control valve testing was completed at 2343 hours and at 0030 hours on June 19, 1997, reactor power ascension began. The reactor reached 100% power at 0540 hours on June 19, 1997 and remained there for the rest of June.

During June 1997, no power operated relief valves (PORV) or primary system safety valve challenges or failures occurred.