

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 5 1	PAGE (3) 1 OF 0 3
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TITLE (4) Missed Technical Specification Surveillance 4.8.1.1.1.a Due to Failure to Complete the Surveillance

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)						
0	1	0	8	8	8	0	0	3	0	0	0	0	3	2	5	Brunswick Unit 2	0 5 0 0 0 3 2 5 1
											0 5 0 0 0						

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
	POWER LEVEL (10) 1 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vi)								
20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(vii)								
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)								
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)								

LICENSEE CONTACT FOR THIS LER (12)

NAME M. J. Pastva Jr., Regulatory Compliance Specialist	TELEPHONE NUMBER 9 1 9 4 5 7 - 2 3 1 5
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS
X	E, K	D, G, N	1, 5, 2	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During Unit 1 power operation at 100%, concurrent with a Unit 2 refuel/maintenance outage, a technical specifications (TS) surveillance required by 0045 hours on 1/8/88 to demonstrate the operability of off-site alternating current (ac) power sources was not completed until 0250 hours on 1/8/88. This surveillance was required due to the inoperability of the units' common ac Diesel Generator (DG) No. 2. At the time, the units' other common DGs (Nos. 1, 3, and 4) were operable. The subject surveillance is required by TS 4.8.1.1.1.a and is accomplished through performance of Periodic Test (PT)-12.8. At 0205 hours on 1/8/88, the subject surveillance was completed and at 0330 hours, DG No. 2 was returned to standby readiness.

This event resulted from failure of the Unit 1 Shift Foreman to adequately track and verify completion of the surveillance. A contributing factor to this failure was distraction of the Shift Foreman's attention due to problems with the Reactor Building ventilation system.

The involved Shift Foreman was disciplined and factors relative to this event were emphasized with the Shift Operating Supervisors (SOSs) by the plant Operations Manager. A memorandum was distributed which called attention of the SOSs to this event and pointed out the vulnerability to such type errors and stressed the importance of strict TS verbatim compliance. This event will be reviewed during Operations on-shift real-time training by 3/31/88.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A (1/7))

Initial Conditions

Unit 1 was operating at 100% power while Unit 2 was in a refueling/maintenance outage. At 0044 hours on January 7, 1988, emergency Diesel Generator (DG) No. 2 (EIIS/EK/DG), which is one of the units' four common 4160 volt (V) alternating current (ac) DGs, was declared inoperable. The inoperability of DG No. 2 required performance of periodic surveillance in accordance with Technical Specification (TS) 4.8.1.1.1.a. This surveillance is performed on the plant electrical breakers to demonstrate operability of off-site ac sources within two hours and at least once per 12 hours thereafter and is accomplished through performance of PT-12.8. The PT had last been performed at 1245 hours on January 7, 1988, and the next required performance of the electrical breaker surveillance was at 0045 hours on January 8, 1988.

Event Description

During the Operations shift turnover at 1900 hours on January 7, 1988, the oncoming Unit 1 Shift Foreman (SF) was informed that the ongoing investigations/repairs to DG No. 2 were nearly complete. The required surveillance had last been performed at 1245 hours on January 7 and was due at 0045 hours on January 8. Following communications with involved plant Maintenance staff, the oncoming SF anticipated the subject work would be completed by approximately 2230 hours and that the required postmaintenance testing of DG No. 2 and the return to operable status would occur by 0030 hours on January 8. The required repairs to DG No. 2 were completed at approximately 2300 hours and plans were initiated to perform postmaintenance testing of the DG.

At 2339 hours, the Reactor Building (RB) ventilation system supply and exhaust fans (EIIS/VA/FAN) tripped concurrent with receipt of the RB ventilation differential pressure HI/LO alarm annunciation. In response, the Unit 1 Control Operator manually started the Standby Gas Treatment (SBGT) System (EIIS/BH/FAN) in order to ensure negative pressure was maintained in the RB.

At 0100 hours on January 8, 1988, the problem affecting the RB ventilation system was resolved and the affected systems were returned to normal lineup.

At approximately 0130 hours on January 8, 1988, the DG start timing and generator loading for postmaintenance testing of DG No. 2 was begun. At approximately 0230 hours, following a discussion with the Shift Operating Supervisor (SOS) regarding testing of DG No. 2, the Shift Foreman realized that the required once per 12 hours electrical breaker surveillance had not been performed within the 12 hours required by TS. This surveillance was immediately commenced and completed by 0250 hours on January 8, 1988.

Required postmaintenance testing of DG No. 2 was satisfactorily completed and at 0330 hours on January 8, 1988, the DG was returned to standby readiness.

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TEXT (If more space is required, use additional NRC Form 368A 2) (17)

Event Investigation

A reconstruction of the circumstances relative to this event was conducted by the involved Shift Foreman in order to determine the root cause of this event. This investigation concluded the failure to perform the subject electrical breaker surveillance within the required 12-hour time frame resulted from failure of the Shift Foreman to adequately track and verify the completion of the surveillance. A major contributor to this failure was the distraction of the Shift Foreman's attention due to the incurred problem with the RB ventilation system. It should be noted that the subject breakers were properly aligned throughout the time period of this event.

Corrective Action

Appropriate disciplinary action has been taken with the involved Shift Foreman regarding this event. The plant Operations Manager made personal telephone communications with the plant SOSs in order to emphasize factors relative to this event. A memorandum was also distributed to the SOSs from the Operations Superintendent which called the attention of the SOSs to these events. The memorandum also stressed the importance of strict verbatim compliance with TS and pointed out the vulnerability to such type errors during unit outage-related activities. This event will be reviewed with the operating shifts as part of the Operations on-shift real-time training to be completed by March 31, 1988.

Event Assessment

An assessment of this event under other reasonable and credible alternative conditions determined this event would not have been more severe.

Prior Similar Events

A review of plant files was performed to identify any prior similar events resulting from a failure of Operations Control Room personnel to perform required TS action statements. This review revealed four prior events which were reported in LERs 2-84-005, 1-86-027, 2-86-025, and 1-87-016.

CP&L

Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

February 5, 1988

FILE: B09-13510C
SERIAL: BSEP/88-0089

10CFR50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-88-003

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz for

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/bvc

Enclosure

cc: Dr. J. N. Grace
Mr. E. D. Sylvester
BSEP NRC Resident Office

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