



Log # TXX-88125
File # 10110
903.8
Ref. # 10CFR50.55(e)

William G. Council
Executive Vice President

February 4, 1988

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION
DOCKET NO. 50-445
SEISMIC CATEGORY I PLATFORMS
SDAR: CP-85-53 (FINAL REPORT)

Gentlemen:

On December 20, 1985, we verbally notified your Mr. R. Hall of a deficiency involving Seismic Category II platforms. On June 19, 1987, we notified you via TXX-6524 that the deficiency only involved a Seismic Category I platform. Our last interim report on this issue was logged TXX-7113, dated December 18, 1987. We have concluded that this condition is reportable under the provisions of 10CFR50.55(e). The required information follows.

DESCRIPTION OF DEFICIENCY

Originally this deficiency was identified as encompassing several platforms which were not fabricated in accordance with prescribed design documents. After further evaluation, we have confined this issue to a single Seismic Category I platform located at elevation 792' in the Reactor Containment Building.

The cause of this deficiency was determined to be inadequate inspection procedures and inadequate construction supervision. The result was an unfinished platform with incomplete QC inspections. The implication of this finding applies to the total population of structural steel with missing inspection documentation and is addressed by CAR 87-012 and associated SDAR CP-87-71, "Missing Welds and Undersized Members."

SAFETY IMPLICATIONS

Failure of this platform during a seismic event could have resulted in a loss of attached safety related instrument lines resulting in loss of associated control/indication functions. The condition represents a significant deficiency in construction requiring extensive evaluation/repair to establish the adequacy of this platform to remain intact during a seismic event.

8802090497 880204
PDR DOCK 05000445
S PDR

IE27
1/0

TXX-88125
February 4, 1988
Page 2 of 2

CORRECTIVE ACTION

The corrective action for this issue is enveloped by the corrective actions for SDAR CP-87-71 and is listed below.

"Seismic Category I and II structural steel assemblies for which inspection are not available will be reworked in accordance with the applicable design drawings. The reworked assemblies will be inspected in accordance with QI-QP-11.14-1, "Inspection of Site Fabrication and Installation of Structural and Miscellaneous Steel." Any discrepancies will be documented and dispositioned using Nonconformance Reports (NCRs)."

This is our final report on this issue. Preventive action, and notification of completion of corrective actions will be provided in response to SDAR CP-87-71, which is scheduled to be submitted no later than March 2, 1988.

Very truly yours,

W. G. Council

By: John W. Beck

John W. Beck
Vice President,
Nuclear Engineering

JCH/grr

c-Mr. R. D. Martin, Region IV
Resident Inspectors, CPSES (3)