JAN 07 1988

Docket No. 50-293

MEMORANDUM FOR: James T. Wiggins, Chief
Reactor Projects Branch No. 3

FROM: Allen R. Blough, Chief
Reactor Projects Section No. 3B

SUBJECT: PILGRIM STATUS REPORT FOR THE PERIOD DECEMBER 17-31, 1987

Enclosed is the Pilgrim bi-weekly status report from the NRC Resident Office at Pilgrim. Three resident inspectors monitored activities at the plant during the report period. A region-based specialist inspector was onsite during the week of December 21, 1987 to observe the conduct of the primary containment integrated leak rate test and to review the test results.

These status reports are intended to provide NRC management and the public with an overview of plant activities and NRC inspection activities. Subsequent inspection reports will address many of these topics in more detail.

Original Signed By:

Allen R. Blough, Chief
Reactor Projects Section No. 3B

Enclosure:
As Stated

OFFICIAL RECORD COPY PILGRIM STATUS REPORT 1/4/88 - 0001.0.0
01/06/88

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cc w/encl:
W. Russell, RI
W. Kane, RI
W. Johnston, RI
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R. Wessman, NRR
R. Bird, Senior Vice President-Nuclear, BECo
K. Roberts, Station Manager, BECO
B. McIntyre, Chairman, Department of Public Utilities
Chairman, Board of Selectmen, Plymouth
Chairman, Board of Selectmen, Carver
Chairman, Board of Selectmen, Duxbury
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The Honorable E. J. Markey
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JAN 07 1988

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Warren/mjd	Blough	Wiggins
1/7/88	1/6/88	1/6/88

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01/06/88

ENCLOSURE

PILGRIM STATUS REPORT FOR THE PERIOD DECEMBER 17-31, 1987

1.0 Plant Status

As of 8:00 a.m., on December 31, 1987, the reactor was in cold shutdown mode with moderator temperature about 95 degrees Fahrenheit. The primary containment head was installed and the integrated leak rate test of the primary containment completed.

2.0 Facility Operations Summary

The plant has been shutdown for maintenance and to make program improvements since April 12, 1986. The reactor core was completely defueled on February 13, 1987 to facilitate extensive maintenance and modification of plant equipment. The licensee completed fuel reload on October 14, 1987. Reinstallation of the reactor vessel internal components and the vessel head was followed by completion of the reactor vessel hydrostatic test.

During this report period the licensee successfully completed the primary containment integrated leak rate test (ILRT) as described in section 3.0.

3.0 Items of Special Interest

Scheduled U.S. Senate Committee Hearing on Pilgrim Restart

In his letter to NRC Chairman Zech on December 17, 1987, Senator Kennedy notified that he will be holding a hearing, as Chairman of the U.S. Senate Labor and Human Resources Committee, concerning the proposed restart of the Pilgrim Nuclear Power Station and its potential impact on the public health. The hearing would be held at the Plymouth North High School in Plymouth, Massachusetts at 7:00 p.m. on January 7, 1988. A panel of private citizens, a panel of public interest groups, a panel of representatives of the Commonwealth of Massachusetts, NRC, and the Federal Emergency Management Agency were invited to participate in the hearing.

Licensee Nuclear Organization Management Realignment

On December 14, and on December 31, 1987, the Boston Edison Co. announced, as part of a planned realignment occurring over the next several weeks, the appointment of the following managers to key management positions in the licensee nuclear organization at Pilgrim Station.

-- Mr. Kenneth L. Highfill was named to assume the new position of Station Director. In this capacity, Mr. Highfill will oversee day to day operation of the Pilgrim Station including plant operations, planning and outage, nuclear training, plant support functions, and administrative services. Mr. Highfill will report directly to Mr. Ralph G. Bird, Senior Vice President-Nuclear.

-- Mr. Robert J. Barrett was named the new Plant Manager. Mr. Barrett will report to Mr. Highfill, the Station Director.

-- Mr. Roy Anderson, currently Deputy Outage Manager, was named to assume the new position of Planning and Outage Manager. Mr. Anderson will report to Mr. Highfill, the Station Director.

-- Mr. Ed Kraft was named to assume the new position of Plant Support Manager. In this capacity, Mr. Kraft will oversee radiological, security, industrial safety and fire protection, and other station support functions. Mr. Kraft will report to Mr. Highfill, the Station Director.

-- Mr. Donald Gillespie, currently Director of Planning and Restart, was appointed to the position of Quality Assurance Department Manager. Mr. Gillespie will assume the position after completing his Senior Reactor Operator training. The Quality Assurance Department Manager reports to Mr. J. E. Howard, Vice President- Engineering.

-- Mr. Frank Famulari, currently Operations Quality Control Group Leader, was named to assume the newly created position of Deputy Quality Assurance Department Manager. Mr. Famulari will report to Mr. Gillespie, and be acting Department Manager until Mr. Gillespie assumes the position after completing the Senior Reactor Operator training.

-- Mr. John F. Alexander was named to assume the position of Operations Section Manager. Mr. Alexander will report to Mr. Barrett, the Plant Manager.

-- Mr. Donald J. Long was named Security Section Manager. Mr. Long will report to Mr. Kraft, the Plant Support Manager.

Performance of the Primary Containment ILRT

On December 21, 1987, the licensee began performance of the primary containment ILRT. Performance of ILRT is a operating license requirement, and it is conducted as specified in the licensee's Technical Specifications. The ILRT is designed to ensure the leak tightness of the primary containment. During the test, the containment and associated plant systems were placed in an alignment as close to that which would exist after an accident as practical. It was then pressurized with air to the full test pressure of 45 pounds per square inch and maintained at this pressure for at least 24 hours. The 24 hour test period was started at 10:15 p.m. on December 21, 1987. During this time, the licensee monitored and measured any leakage. The total allowable leakage limits are described in NRC regulations and the licensee's Technical Specifications.

The preliminary licensee test results indicated a successful test, with measured leakage slightly greater than 20 percent of the allowable leakage. A primary contributor to the observed leakage was identified as a drywell pressure transmitter instrument piping cap which had not been fully tightened. Additionally, a water leak was identified in the drain piping attached to the high pressure coolant injection turbine exhaust line. A regional specialist inspector was onsite during the ILRT to review the adequacy of the test procedure and to observe the conduct of the test. Upon completion of the inspector's review of the ILRT results, the inspection report 50-293/87-58 will be issued documenting the inspector's findings.

Termination of an NRC Order Modifying License

In November 1984, as a result of significant deficiencies in the Radiological Controls Program at Pilgrim Station, an NRC Order Modifying License was issued. The Order required the following:

-- Performance of an independent contractor assessment of the Radiological Controls program,

-- Submission of copies of the assessment, to the Regional Administrator,

-- Development and submittal to the Regional Administrator of an Interim Plan for achieving adequate management oversight of radiological work in progress,

-- Development and submittal to the Regional Administrator of a Radiological Improvement Plan for upgrading the Radiological Controls Program.

Multiple program inspections were conducted following issuance of the Order to monitor its implementation and evaluate the adequacy and effectiveness of Radiological Controls Program upgrades. In October, 1986, a special Health Physics Appraisal Inspection (50-293/86-19) was conducted for the purpose of determining whether the licensee had fully established and implemented an effective Radiation Protection Program in response to the Order. However, deficiencies identified in the areas of communication and working relationships, personnel accountability, and corrective actions precluded closure of the Order at that time. A second special inspection (50-293/87-43) was conducted during the period on November 2-5, 1987, to review the licensee's actions on the remaining program deficiencies.

In his letter to the licensee on December 22, 1987, Mr. William T. Russell, Regional Administrator-NRC Region I, notified the licensee that the Order was terminated based on the results of the special inspection (50-293/87-43) in conjunction with the results of the multiple program inspections performed between the two special inspections which indicated that all terms of the Order have been satisfactorily completed.

4.0 Emergency Notification System (ENS) Report

During this period, the licensee made one report to the NRC pursuant to 10 CFR 50.72:

On November 17, 1987, at 11:05 a.m., the licensee experienced a spurious automatic closure of the inboard primary containment isolation valve on the reactor water cleanup (RWCU) system suction line. Investigation by the licensee indicated that the technicians replacing temperature switches in the RWCU high area temperature isolation logic instrument system inadvertently grounded a wire which had been lifted during the replacement work. Grounding the wire resulted in a blown logic power fuse and deenergization of this portion of the logic caused the valve to automatically close. The fuse was replaced and the test subsequently completed.

5.0 NRC Staff Status During the Period

The inspection staff at Pilgrim during the report period consisted of the following:

Clay Warren --- Senior Resident Inspector

Jeffrey Lyash --- Resident Inspector

Tae Kim --- Resident Inspector

In addition, one regional specialist inspector was onsite during the week of December 21, 1987 to observe the primary containment ILRT and to review the test results. The results of this inspection will be documented in NRC inspection report 50-293/87-58.

Mr. Allen R. Blough, the chief of the Reactor Projects Section 3B (Region I), was onsite during December 21-22, 1987. Mr. Blough toured the plant with the resident inspector and attended a resident inspector's exit interview with licensee management. At the exit interview, the results of the inspection 50-293/87-50, which covered the period of October 27 - December 6, 1987, were discussed with the licensee management.