

NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Grand Gulf Nuclear Station - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 1 6				PAGE (3) 1 OF 0 3								
TITLE (4) Inadvertent Division II LOCA Signal During Restoration from Surveillance Procedure																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES NA				DOCKET NUMBER(S) 0 5 0 0 0									
1	2	0	8	8	7	8	7	0	2	3	0	0	1	0	7	8	8	0	5	0	0	0
OPERATING MODE (9) 5		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11)																				
POWER LEVEL (10) 0 1 0 1 0		20.402(b)				20.406(a)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)								
		20.406(a)(1)(i)				50.36(a)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)								
		20.406(a)(1)(ii)				50.36(a)(2)				<input type="checkbox"/> 50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)								
		20.406(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)												
		20.406(a)(1)(iv)				50.73(a)(2)(B)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)												
		20.406(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(x)												
LICENSEE CONTACT FOR THIS LER (12)																						
NAME Paul M. Different/Plant Licensing Engineer										TELEPHONE NUMBER 6 0 1 4 3 7 - 2 1 6 7												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR						
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO										

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On December 8, 1987 while performing the restoration section from surveillance procedure "RHR Containment Spray Initiation Logic System Functional Test," a licensed reactor operator directed Instrumentation and Control (I&C) technicians to remove a temporary Emergency Core Cooling System (ECCS) test switch prior to resetting the Residual Heat Removal (RHR) "B"/RHR "C" initiation logic. This was not the correct sequence specified in the approved procedure. This personnel error caused an inadvertent Division II Loss of Coolant Accident (LOCA) signal and actuated the Combustible Gas Control System "B" train.

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NRC Form 305A
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMR NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Grand Gulf Nuclear Station - Unit 1	0 5 0 0 0 4 1 6	8 7	— 0 2 3	— 0 1 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 305A's) (17)

A. REPORTABLE OCCURRENCE

On December 8, 1987 at 0340 during restoration steps of a surveillance test procedure, procedural actions were performed out of sequence due to personnel error. These actions caused an inadvertent Division II Loss of Coolant Accident (LOCA) signal which actuated the Combustible Gas Control System. This incident is reportable pursuant to 10CFR50.73(a)(2)(iv).

B. INITIAL CONDITION

At the time of the event, the plant was shutdown for a scheduled refueling outage with Division II inoperable for maintenance work and surveillance testing. Reactor Water Cleanup System (RWCU) pumps were in operation with the reactor vessel head removed and upper cavity flooded.

C. DESCRIPTION OF OCCURRENCE

On December 8, 1987 during restoration from surveillance procedure "RHR Containment Spray Initiation Logic System Functional Test," a licensed reactor operator instructed an Instrumentation and Control (I&C) technician to remove a temporary Emergency Core Cooling System (ECCS) test switch per step 5.2.33 prior to resetting the Residual Heat Removal (RHR) "B"/RHR "C" initiation logic per step 5.2.32.i. This sequence was not the correct sequence of steps for restoration and resulted in an inadvertent Division II LOCA signal and an ESF actuation of the Combustible Gas Control System "B" train. No ECCS injected into the vessel during the event.

Upon actuation, automatic actions were verified by the operators. When the operators determined the cause of the actuation, each of the affected systems was returned to its normal standby lineup. Plant systems were restored to normal lineup at 0400 on December 8, 1987.

D. APPARENT CAUSE

The cause of this event is personnel error due to performance of an action out of sequence as specified by the approved procedure. The surveillance had been successfully run and was in the final steps of restoration when the operator sent I&C technicians to remove the test switch prior to ensuring step 5.2.32.i had been completed. The operator was aware of the need to reset the RHR "B"/RHR "C" initiation logic, but forgot to caution the I&C technician before giving the instruction resulting in the inadvertent actuation.

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NRC Form 365A (9-83)		LICENSEE EVENT REPORT (LER) TEXT CONTINUATION				U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88			
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (5)			PAGE (3)		
Grand Gulf Nuclear Station - Unit 1		0 5 0 0 0 4 1 6		8 7	-	0 2 3	-	0 1 0	0 3 OF 0 3
TEXT (If more space is required, use additional NRC Form 365A's) (17)									
<p>E. SUPPLEMENTAL CORRECTIVE ACTION</p> <p>The licensed operator was disciplined and counseled on his failure to follow approved procedures verbatim.</p> <p>F. SAFETY ASSESSMENT</p> <p>No safety consequence resulted from the actuation since all systems functioned as required and Division II was not being relied upon for any of its safety functions. The event resulted in an unnecessary challenge to plant safety systems.</p>									

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OLIVER D. KINGSLEY, JR.
Vice President
Nuclear Operations

January 7, 1988

U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-29
Inadvertent Division II LOCA
Signal During Restoration
from Surveillance Procedure
LER 87-023-00
AECM-88/0005

Attached is Licensee Event Report (LER) 87-023-00 which is a final report.

Yours Truly,

ODK:bms
Attachment

cc: Mr. T. H. Cloninger (w/a)
Mr. R. B. McGehee (w/a)
Mr. N. S. Reynolds (w/a)
Mr. H. L. Thomas (w/o)
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