



*** SAFEGUARDS INFORMATION ***

Docketing

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SEP 13 1993

Docket No. 50-219

Mr. John J. Barton
Vice President and Director
Oyster Creek Nuclear Generating Station
P. O. Box 388
Forked River, NJ 08731

Dear Mr. Barton:

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION -
OPERATIONAL SAFEGUARDS RESPONSE EVALUATION
(TAC NO. M86769)

This letter conveys the results and conclusions of the Operational Safeguards Response Evaluation (OSRE) conducted by the NRC's Office of Nuclear Reactor Regulation at the Oyster Creek Nuclear Generating Station from July 12 through 15, 1993. The OSRE team was composed of NRC personnel assisted by members of the U.S. Army Special Forces. One purpose of the OSRE program is to evaluate a licensee's ability to respond in a contingency to an external threat by focusing on the interactions between operations and security in establishing priorities for protection of equipment and on the defensive strategies used. The OSRE also includes a safety/safeguards interface review to assure that safeguards measures do not adversely affect safe operation of the facility.

The OSRE conclusions are documented in the enclosed report (Part I, Operational Safeguards Readiness Review; and Part II, Safety/Safeguards Interface Review). This enclosure, which contains safeguards information of a type specified in 10 CFR 73.21, will not be placed in the Public Document Room, and must be protected against unauthorized disclosure.

The drills, exercises, and demonstrations observed by the team and the results of interviews conducted by the team indicated weaknesses in three areas that should be addressed and strengths in two other areas. The weaknesses that warrant your attention are identified in the report as concerns. The team also concluded that effective provisions were in place to assure that safeguards measures did not adversely affect the safe operation of the facility.

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Mr. John J. Barton

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The weaknesses described in the report, particularly those in 2.1.3.1, concern a long-standing open issue regarding your protection program for certain safety related equipment. This issue was identified in a NRC Region I inspection report, 50-219/86-39, dated March 9, 1987, and in a Regulatory Effectiveness Review Report forwarded to GPU Nuclear (GPUN) on August 19, 1988. In a meeting on January 3, 1990, GPUN described reactor building access control modifications and informed the NRC that the modifications would be completed by early 1992. An April 1990 SALP report noted a licensee commitment to "include resolution of the boundary issue in its long-range plans and to correct vagueness in the plan revision." However, a June 5, 1992 update to the Oyster Creek Integrated Schedule deferred the modification to 1997. In a letter dated September 14, 1992, GPUN proposed canceling the modification based on a number of factors described as augmenting protection against the design basis threat. In a letter dated November 18, 1992, we informed you that, to complete our review of your proposal, we would evaluate your contingency response capability for protecting related equipment as part of an OSRE. Based on the findings of the team, we believe that progress has been made in resolving the reactor building issue, however, weaknesses still exist that should be corrected.

The enclosed report does not convey any new regulatory requirement. Its findings have been considered with respect to your ability to meet the general performance objective and requirements of 10 CFR 73.55(a). We request that you review the report and provide a response to this office within 45 days after receiving this letter. Your response should specifically address each of the concerns identified in the report and describe the corrective actions you intend to take to resolve these concerns along with a schedule for implementation. After receiving your response we plan to schedule a follow-up visit to the site if necessary to complete our review and resolve any remaining open issues.

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This letter affects one respondent and, therefore, is not subject to the Office of Management and Budget review under Public Law 96-511.

Sincerely, **Original** Signed by

Steven A. Varga, Director
 Division of Reactor Projects I/II
 Office of Nuclear Reactor Regulation

Enclosure:
 Operational Safeguards Response Evaluation
 (Part I, Operational Safeguards Readiness Review;
 Part II, Safety/Safeguards Interface Review)

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cc w/enclosure:
 David J. Vito, SRI
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Oyster Creek Nuclear
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cc:

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