Attachment 1

Technical Specification 3/4.2.3 - RCS Flowrate and Nuclear Enthalpy Rise Hot Channel Factor

Proposed Change

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PDR ADOCK 05000

Revise Figure 3.2-3 to lower the minimum Reactor Coolant System flowrate to 383,500 gpm for operation at or below 90% Rated Thermal Power. This value corresponds to 95% of thermal design flow plus measurement uncertainties of 3.5%.

Justification and Safety Analysis

Preliminary measurements of total RCS flow at McGu're Unit 1 indicate that the measured flow may be less than the Thermal Design Flow used in the plant accident analyses. The Technical Specifications allow a finite time period in which to restore RCS flow to above the minimum requir a value; however the time period is based on the assumption that the low flow is due to time limited local effects, e.g. grid voltage/frequency dips. The indications at McGuire are not believed to be due to grid or flow fluctuations, but rather the results of actions taken to increase core bypass flow to lower upper head temperature to T_c .

In the event the RCS flow is actually verified to be less than Thermal Design Flow, the plant Technical Specifications are quite specific in action to be taken. However the Technical Specifications do not recognize the possibility of a long term reduction in flow, nor the relationships between flow and DNB or flow and core power which allow various trade-offs. These trade offs allow continued operation at some reduced flow and a corresponding reduction in allowed maximum power.

It is widely recognized that there are relationships between core power, flow, and DNB as noted below:

00	Flow DNB	•	17	(Eq	•	1)
00	Power		1%	(Eq		2)

thus the relationship between Power and Flow is:

a Power	1%	12-	23
3 Flow	1.8%	(29.	2)

As a result of the above noted relationships and a conservative assumption (based on current indications) that the verified RCS flow will be no lower than 0.95 of Thermal Design Flow it is proposed that McGuire Unit 1 be operated at a maximum power level of 90% of Rated Thermal Power (3070 MWth core power) in the event that measured flow is less than Thermal Design Flow (plus measurement uncertainty).

This reduction in core power is the equivalent of an RCS flow increase of ap-

proximately 13% in terms of margin to DNB. Since the expected flow deficit is 5% or less, the actual result in the power reduction is an increase of approximately 13% in terms of margin to DNB.

Operation of the plant within this power restriction results in no increase in T_{avg} thus there is no temperature impact in terms of margin to DNB.

McGuire Unit l's Technical Specification limits and accident analyses results have been evaluated to determine the impact of the RCS flow and power reductions. In all cases sufficient margin exists to allow plant operation at the reduced power level. No Technical Specification limits require modification, including core limits, OTAT, OPAT, and Power Range Neutron Flux - High, High setpoint.

The core limits remain the same due to the increased margin to DNB afforded by the power reduction and interpretation that they are now valid for 3070 MWth instead of 3411 MWth. This implies that 3070 MWth should be considered to be 1.0 in fraction of Rated Thermal Power for Figure 2.1-1. With no change in the core limits the OTAT and OPAT trip setpoints remain unchanged. Utilizing the latest Westinghouse data, the uncertainty in the instrumentation for the Power Range Neutron Flux - Pigh trip function is 4.7% span (or 5.6% RTP). With a normal assumption of reactor trip at 109% RTP the uncertainty analysis verifies that a trip will take place at 109% RTP plus 5.6% RTP, or 114.6% RTP. A 5% reduction in RCS flow requires a trip at 115.2% RTP, thus adequate margin exists in the instrumentation such that no change in the nominal setpoint is necessary.



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FIGURE 323 RISTOTAL FLOMPATE VERSUER, AND R2-FOUR LODIS IN OPERATION



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FIGURE 233 RCS TOTAL FLOMMATE VERSUS Ry MDR2-FOUR

LOUS IN OTERATION

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FIGURE 3.2.3 RCS TOTAL FLOWRATE VERSUS R1 AND R2-FOUR LOOPS IN OPERATION

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DUKE POWER COMPANY

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WILLIAM O. PARKER, JR. JOF PRESIDENT STEAM PRODUCTION

November 11, 1981

TELEPHONE: AREA 704 373-4083

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief Licensing Branch No. 4

Re: McGuire Nuclear Station Docket Nos. 50-369

Dear Mr. Denton:



The proposed amendment is considered to be a Class III amendment pursuant to 10CFR 170.22. Therefore, enclosed is a check in the amount of \$12,300.

Very truly yours, . W. Tau William O. Parker, Jr.

GAC/smh

Attachment

cc: M. J. Graham Resident Inspector McGuire Nuclear Station

Mr. James P. O'Reilly, Director U. S. Nuclear Regulatory Commission Region II 101 Marietta Street, Suite 3100 Atlanta, Georgia 30303



Mr. Harold R. Denton, Director Page 2 November 11, 1981

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this revision to the McGuire Nuclea: Station Technical Specifications, Appendix A to License No. NPF-9; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

Unen Q. Tarker . William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 11th day of November, 1981

Aue C. Sherrill Notary Public

My Commission Expires:

September 20, 1914