### U. S. NUCLEAR REGULATORY COMMISSION

#### REGION III

Report No. 50-461/87033(DRS)

Docket No. 50-461

License No. NPF-62

Licensee: Illinois Power Company 500 South 27th Street Decatur, IL 62525

Facility Name: Clinton Nuclear Power Station, Unit 1

Inspection At: Clinton Site, Clinton, Illinois

Inspection Conducted: September 4, 1987 through Januar, 14, 1988

Inspectors: A. M. Bongiovanni

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S. Stasek

Approved By: M. P. Phillips, Chief

Operational Programs Section

Inspection Summary

Inspection on September 4, 1987, through January 14, 1988 (Report No. 50-461/87033(DRS))

Areas Inspected: Routine, unannounced safety inspection to review startup test results. (72532, 72301)

Results: Within the one area inspected, no violations or deviations were identified.

## DETAILS

#### 1. Persons Contacted

# Illinois Power Company

- \*J. W. Wilson, Manager, Clinton Power Station
- \*R. E. Campbell, Manager, QA \*R. D. Freeman, Manager, NSED
- \*J. S. Perry, Manager, Nuclear Program Coordination
- \*R. E. Wyatt, Manager, Nuclear Training
- \*D. L. Holesinger, Assistant Manager, Plant Staff \*A. M. MacDonald, Director, Nuclear Program Assessment
- \*A. L. Ruwe, Director, Outage Maintenance Support
- \*R. A. Schultz, Director, Planning and Programming \*J. D. Weaver, Director, Licensing
- \*K. A. Baker, Supervisor, I&E Interface

# 2. Startup Test Results Evaluation

The inspectors reviewed the results of the startup test procedures listed below to verify that all test changes were identified and approved in accordance with administrative procedures; all test deficiencies were appropriately resolved, reviewed by management, and retested as required; test results were evaluated by appropriate engineering personnel and specifically compared with acceptance criteria; data was properly recorded, signed, dated, and documented as test deficiencies if out of tolerance, and test results were approved by appropriate personnel:

STP-06-H	SRM Performance and Control Rod Sequence
STP-11-H	LPRM Calibration (Response Check)
STP-14-H	RCIC System
STP-53-H	Drywell Atmosphere Cooling
STP-05-0	Control Rod Drive System
STP-10-0	IRM Performance - SRM/IRM Overlap
STP-10-1	IRM Performance - IRM/APRM Overlap
STP-19-1	Core Performance (TC #1)
STP-22-1	Pressure Regulator Test (TC #1)
STP-05-2	Control Rod Drive System (TC #2)

<sup>\*</sup>Denotes those personnel listed above who attended the exit interview on January 14, 1988.

Core Performance (TC #2) STP-19-2 Pressure Regulator Test (TC #2) STP-22-2 Shutdown From Outside The Control Room STP-28-2 Core Performance (TC #3) STP-19-3 STP-22-3 Pressure Regulator Test (TC #3) STP-19-4 Core Performance (TC #4) STP-19-5 Core Performance (TC #5) Control Rod Drive System (TC #6) STP-05-6 Core Performance (TC #6) STP-19-6 STP-53-6 Drywell Atmosphere Cooling (TC #6)

Inspector review of STP-19-2 and STP-19-4 had not been completed at the end of the inspection. Completion of which will, therefore, be documented in a subsequent inspection report.

All procedures reviewed appeared acceptable. However, the inspectors had the following comments with respect to two procedures:

- a. With respect to STP-53-6, the inspector noted that Appendix B listed 18 permanent plant instrument loops used to obtain temperature data during the test as well as the last calibration date of each. The test was conducted during September/October 1987. However, some of the subject instruments had calibration dates as early as September-November 1985. When questioned about the periodicity for recalibration of these instruments, the licensee responded that they were categorized in accordance with station administrative procedures and designated "Category D" items. It was explained that "Category D" items had no requirement for periodic recalibrations and were, therefore, only recalibrated upon request on a case-by-case basis. The inspector will further evaluate the licensee's program in this area and the potential impact on the specific data obtained during the performance of this test. This is considered an open item (461/87033-01(DRS)).
- b. With respect to STP-53-H, the inspector noted that acceptance criteria for the maximum allowable temperature in containment had been changed for several areas via an approved Test Change Notice (TCN). The largest change observed involved temperature element C217 located near the top of the bioshield. The original maximum allowable temperature for this location was 150°F which was consistent with the acceptance criteria specified in Chapter 14 of the Final Safety Analysis Report (FSAR). The licensee initially increased the allowable value for this location to 180°F. When it

was subsequently determined that the new value also could not be met, the allowed value was again increased, this time to  $225^{\circ}F$ . The data recorded during the performance of the test indicated the maximum temperature reached at the C217 location was 210°F. The licensee performed a safety evaluation pursuant to 10 CFR 50.59 and determined that a FSAR change was required. Change #176 was prepared to document this but at the time of the inspection, the change had not been submitted for review and final incorporation into the FSAR. Further evaluation by the inspector of the acceptability of increasing the maximum allowable temperature limits in containment is required. This will be tracked as an open item (461/87033-02(DRS)).

No violations or deviations were identified.

# 3. Open Items

Open items are matters which have been discussed with the licensee, which will be reviewed further by the inspector, and which will involve some action on the part of the NRC or licensee or both. Two open items disclosed during the inspection were discussed in Paragraphs 2.a and 2.b.

### 4. Exit Interview

The inspectors met with licensee and contractor representatives denoted in Paragraph 1 during and at the conclusion of the inspection on January 14, 1988. The inspectors summarized the scope and results of the inspection and discussed the likely content of this inspection report. The licensee acknowledged the information and did not indicate that any of the information disclosed during the inspection could be considered proprietary in nature.