

Public Service of New Hampshire

New Hampshire Yankee Division

NYN-88009

January 25, 1988

United States Nuclear Regulatory Commission Washington, DC 20555

Attention: Document Control Desk

References: (a) Facility Operating License No. NPF-56, Docket No. 50-443

- (b) PSNH Letter (SBN-1086), dated June 4, 1986, "Inservice Testing (IST) of Pumps and Valves (SER Outstanding Issue No. 4)," G. S. Thomas to V. S. Noonan
- (c) PSNH Letter (SBN-1136), dated June 23, 1986, "Inservice Testing (IST) of Pumps and Valves (SER Outstanding Issue No. 4)," G. S. Thomas to V. S. Noonan
- (d) PSNH Letter (SBN-1144), dated June 25, 1986, "Inservice Testing (IST) of Pumps and Valves (SER Outstanding Issue No. 4) Relief Request No. 46," G. S. Thomas to V. S. Noonan
- (e) PSNH Letter (NYN-87081), dated June 17, 1987, "Inservice Testing (IST) of Pumps and Valves (Changes to Revision 1 of the Inservice Testing Program)," G. S. Thomas to NRC

Subject: Inservice Testing (IST) of Pumps and Valves (Changes to Revision 1)

Gentlemen:

Reference (e) provided changes to Revision 1 of the Seabrook Station Inservice Testing (IST) Program for pumps and valves. These changes were reviewed by the NRC Staff and their contractor, EG&G. On December 18, 1987, members of the NRC Staff, EG&G, and New Hampshire Yankee (NHY) discussed, by telephone, several concerns that were identified during the review. The enclosed material provides the changes to the IST program that are necessary to resolve the Staff concerns.

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Should you have any further questions regarding this matter, please contact our Bethesda office (Mr. R. E. Sweeney) at 301-656-6100.

Very truly yours,

George S. Thomas

Enclosure

cc: Mr. William T. Russell
Regional Administrator
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Mr. Victor Nerses, Project Manager Project Directorate I-3 Division of Reactor Projects United States Nuclear Regulatory Commission Washington, DC 20555

Mr. Antone C. Cerne NRC Senior Resident Inspector Seabrook Station Seabrook, NH 03874 ENCLOSURE TO NYN-88009

Relief Request:

P-3

Pumps:

CC-P-11A, CC-P-11B, CE-P-11C, CC-P-11D, RH-P-8A, RH-P-8B, SW-P-41A, SW-P-41B, SW-P-41C, SW-P-41D, SW-P-110A and SW-P-110B

Code Class:

2 and 3

Function:

Pumps required to perform a function in shutting down the reactor or in mitigating the consequences of an accident, and are provided with an emergency power source.

Test Requirements:

Table IWP-3100-2 specifies the allowable ranges of inservice test quantities in relation to reference values. This table limits the acceptable performance of the differential pressure (Δ P) for each pump to 103 percent (1.03 Δ Pr). If the differential pressure of any pump exceeds this limit, the pump shall be declared inoperable and not returned to service until the cause of the deviation has been determined and the condition corrected.

Basis for Relief:

The requirement to declare a pump inoperative when pump performance exceeds the reference value by 3 percent is impractical and is not clearly indicative of pump degradation for the following reasons:

- a. The 3 percent limitation is overly restrictive when compared to the total accuracy of the instrumentation used to gather the test data. In order to consistently remain below the 3 percent limitation, extremely lower instrument loop accuracies than those required by Table IWP-4110-1 would have to be established. This is particularly evident when testing pumps that have high flow rates and low discharge head characteristics such as the pumps listed above.
- b. Power plant operating systems were not designed to provide laboratory type conditions required to meet the 3 percent limitation. The primary component cooling water (CC) and service water (SW) systems require the use of large butterfly valves using remote manual control to throttle large volumes of water to the reference flow rate. Normally flow rates can only be established to + 100 gpm of the specified reference flow. A deviation of flow of 200 gpm can significantly affect measured test quantities.

Basis for Relief: (continued)

- c. Reference values are specific sets of values determined by measuring or observing pump performance when a specific pump is known to be performing its required function acceptably. Merely exceeding the 3 percent limitation is not a clear signal of pump degradation. It may signify that the reference value is at the lower side of the statistical scatter of the test data in comparison with other periodic test data.
- d. For the pumps listed, the difference between the differential pressure reference value and 103 percent required action value have been in the range from 1.7 to 5.7 psid. This is extremely restrictive and is easily exceeded by any combination of statistical scatter, instrument inaccuracy and minor flow variations. In the past, these pumps have been declared inoperable due to high differential pressure readings exceeding the allowed range by less than 0.5 psid. Further evaluation of these conditions has shown there has been no pump degradation. Furthermore, since an actual increase in pump differential pressure is not indicative of degraded pump performance, it is not necessary to maintain such a strict upper limit.
- e. Relief from the 3 percent limitation will provide an acceptable level of quality and safety and will not endanger the public.

Alternate Testing:

Pumps shall be tested in accordance with Subsection IWP with the following exceptions:

- a. The Required Action Range (High) will be a value greater than 1.10 Δ Pr.
- b. The Alert Range (High) will be a value greater than 1.05 \triangle Pr.

Relief Request:

P-4

Pumps:

SW-P-110A, SW-P-110B, SW-P-41A, SW-P-41B, SW-P-41C and SW-P-41D

Code Class:

3

Function:

Pumps required to perform a function in shutting down the reactor or in mitigating the consequences of an accident, and are provided with an emergency power source.

Test Requirements:

IWP-4200 requires direct pressure measurements. Table IWP-4110-1 acceptable instrument accuracy for pressure measurement.

Basis for Relief:

- a) The above listed pumps are vertical turbine pumps with no direct means to obtain the inlet pressure measurements as required by IWP-4200.
- b) Plant installed level instrumentation is accurate to \pm 0.5% which is within the requirements of Table IWP-4110-1, but total loop accuracy is \pm 2.5% which exceeds the requirement of Table IWP-4110-1.

Alternate Testing:

The inlet pressure shall be calculated based on the water level above the pump inlet using existing plant instrumentation to measure pump suction pressure. Relief Request:

V-48

Valves:

FW-V99, FW-V216, FW-V357 and CO-V-340

Category:

C

Code Class:

3 and NNS

Function:

Active

Test Requirements:

IWV-3530 (3 months)

Basis for Relief:

Full flow through these normally closed check valves quarterly during power operations would require establishing emergency feedwater flow to the steam generators and would unnecessarily introduce cold water into the steam generators causing thermal shock to the feed nozzles. This is the only full flow path for these valves. Thermal shocking of the feed nozzles could lead to their premature failure. Also, introducing the required flow to full-stroke exercise these valves could cause feedwater control problems during operation which could lead to a plant trip.

Alternate Testing:

All of these valves shall be full stroke exercised during cold shutdowns. CO-V-340 shall be partially stroked on a quarterly basis.

FIGURE 5.3
VALVE TESTING REQUIREMENTS

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FIGURE 5.3 VALVE TESTING REQUIREMENTS

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FIGURE 5.3
VALVE TESTING REQUIREMENTS

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FUNCTION	A	Α	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A
CODE CLASS	2	2	2	2	2	12	2	2	2	2	2	2	2	2	2	2	2	2	2	2	2
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