LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MARCH 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

IE24
1/1

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761, extension 705.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

March 1-31

March 1, 0001 hours	Unit One entered March with the Reactor Critical and the generator on-line at 1080 MWe.
March 12, 0630 hours	Begin shutdown for second refuel outage.
March 13, 0150 hours	Generator taken off line at 130 MWe and Reactor manually scrammed.
March 13, 0155 hours	LaSalle Unit One Refuel Outage No. 2 begins.
March 31, 2400 hours	Unit One in Second Refuel Outage.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility License or Technical Specification.

   Amendment number: 54
   Amendment changes locations of breakers for RHR shutdown cooling isolation valve (1E12-F009).

   Amendment number: 55
   Jet pump operability surveillance.

   Amendment number: 56
   Administrative change to include Nuclear Safety Review of all reportable occurrences per GL 83-43.

2. Changes to procedures which are described in the Safety Analysis Report.

   There were no changes made to procedures which are described in the Safety Analysis Report.

3. Tests and Experiments not described in the Safety Analysis Report.

   There were no tests or experiments conducted which are not described in the Safety Analysis Report.

4. Corrective Maintenance of Safety Related Equipment.

   The following table (Table 1) presents a Summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

5. Completed Safety Related Modifications.

   The following Table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L70398	Fuel pool leakage line isolation valve	Metal chips in seat	Valve not seating, allowing enough flow to pick up alarm	Replaced disc and lapped seat
L72772	Div. II swgr heat removal temp controller 1TIC-VX008	Bad love controller	Improper indication	Replaced controller
L74204	LPCS pump cooler valve 1DG035	Faulty 74 relay	Relay not picking up	Replaced relay
L74444	"0" D/G Freq. meter 0SI-DG02BB	Meter out of calibration	Indicating low frequency	Recalibrated
L76208	Reactor vessel Lo Lo water level isolation switch 1B21-N02J3B	Faulty switch	Improper indication during calibration	Replaced switch
L76280	RCIC suction pressure indicator	Indicator out of calibration	Improper indication	Recalibrated pressure indicator
L76354	Temperature switch 0TIC-VD005	Switch out of calibration	Spurious high temperature alarms	Recalibrated switch
L76358	Blocking diode at panel 0DG03J	Blocking diode shorted	Improper operation	Replaced diode
L76420	Control power relay 0VD01C	Faulty Relay	Contacts 61 and 62 not making when deenergized	Replaced Relay
L76434	"B" D/G voltmeter	Meter out of calibration	Meter reading 80 - 100 volts too high	Recalibrated

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L76710	B/C RHR CSCS area cooler	Contact linkage improperly adjusted	Cooler would not autostart	Adjusted contact linkage
L76819	"F" IRM	Loose connection under vessel	Spurious half scrams	Repaired connection
L78927	APRM "D" -20 VDC power supply	Teflon bushing dirty	Power supply shorting out	Cleaned bushing
L79036	Div. III Battery Charger	Bad circuit board	Charger tripping off spuriously	Replaced board

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER:	A brief synopsis of incorporated modification objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT ONE

M-1-0-86-009: Modification of power supply circuitry to Technical Support Center doors in order to provide power from the same MCC and diesel generator backup.

M-1-1-87-016: Upgrade cable tray in Auxiliary Building for addition of ARI/MSIV.

M-1-1-87-083: Snubber reduction on subsystem 1VG01 and 1VG03 per snubber reduction program.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, March 1, 1988 through March 31, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
88-001-00	2/22/88	RCIC High Reactor Water Level Switch Failure.
88-002-00	3/16/88	30 Day - Type B and C Total Leakage Exceeded
88-003-00	3/25/88	30 Day - 2 INOP IRMs During Core Alterations

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE April 10, 1988
COMPLETED BY S. J. Samolinski
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: March 1988 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 0
4. REASONS FOR RESTRICTION (IF ANY): Refuel Outage

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	289.8	1729.8	21774.3
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	1642.0
7.	HOURS GENERATOR ON LINE	289.8	1729.8	21159.8
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	848976	5252088	57086386
10.	GROSS ELEC. ENERGY GENERATED (MWH)	286164	2227148	19194407
11.	NET ELEC. ENERGY GENERATED (MWH)	270406	2152430	18251071
12.	REACTOR SERVICE FACTOR	39.0%	79.2%	58.4%
13.	REACTOR AVAILABILITY FACTOR	39.0%	79.2%	62.8%
14.	UNIT SERVICE FACTOR	39.0%	79.2%	56.8%
15.	UNIT AVAILABILITY FACTOR	39.0%	79.2%	56.8%
16.	UNIT CAPACITY FACTOR (USING MDC)	35.1%	95.1%	47.3%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	33.7%	91.4%	45.4
18.	UNIT FORCED OUTAGE RATE	0.0%	0.0%	13.2%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
N/A

20. IF SHUT DOWN IS AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
Unit One is presently in its second refuel outage. Estimated date of startup is June 26, 1988.

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: April 10, 1988
COMPLETED BY: S. J. Samolinski
TELEPHONE: (815) 357-6761
MONTH: MARCH, 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	1043	17.	- 18
2.	1038	18.	- 19
3	1032	19.	- 13
4.	1007	20.	- 13
5.	994	21.	- 12
6.	991	22.	- 8
7.	985	23.	- 7
8.	973	24.	- 9
9.	968	25.	- 9
10.	959	26.	- 9
11.	954	27.	- 8
12.	535	28.	- 8
13.	- 3	29.	- 5
14.	- 20	30.	- 9
15.	- 20	31.	- 4
16.	- 19		

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE April 10, 1988
COMPLETED BY S. Samolinski
TELEPHONE (815)357-6761

REPORT MONTH MARCH, 1988

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
4	3/13/88	S	454.2	C	1	Second refuel outage

DOCUMENT ID 0043r/0031r

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety Relief Valves actuated during the reporting period March 1 through March 31, 1988.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
0-58-88	0DG01P	Lubricate coupling
0-59-88	0DG01K	Inspection, replacement of oil circ. pump motor
0-60-88	0DG01K	Surveillance LMS-DG-01
0-61-88	ACB 1413, 2413 0DG01K	Electrical Surveillance
0-63-88	0DG01K	BOP Calibrations
0-73-88	"0" D/G	Fix oil leak
1-04-88	1E22-C302B Petter diesel A/C	Prevent operation
1-131-88	1DG090A	Leave power off air dryer
1-174-88	HPCS	Installation of CY flushing spoolpiece
1-182-88	"1B" D/G	Surveillance LES-DG-102
1-183-88	"1B" D/G	Replace stator temperature indicator
1-184-88	"1B" D/G	Repair 1D0014
1-185-88	"1B" D/G	Surveillance LMS-DG-01
1-186-88	"1B" D/G	Clean coolant sightglass
1-187-88	"1B" D/G	Hydrolaze D/G cooler
1-188-88	"1B" D/G	Repack 1E22-F319
1-191-88	1DO060 for 1DG04T	Drain HPCS day tank
1-209-88	1E12-F092B/C	Temporary removal of conduit support
1-246-88	HPCS	Relay logic test
1-266-88	1DO02T	Prevent draining or filling of tank
1-267-88	1E12-F063B	Fill reactor vessel

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-273-88	1E51-F064 Steam to RHR	Determinate
1-284-88	"1B" D/G	Surveillance LES-GM-129
1-285-88	1E51-F064	Repair valves
1-287-88	LPCS CY spool piece	Install spool piece
1-291-88	RCIC drain pot	Repair 1E51-F053
1-292-88	1E22-F023	Rework valve
1-293-88	1E22-F023	Surveillance LES-EQ-112
1-294-88	HPCS pump	Disassemble motor for inspection and maintenance
1-295-88	HPCS pump	Motor inspection
1-296-88	HPCS water leg pump	Replace min. flow orifice Repair discharge check valve
1-297-88	1E22-F010	Inspect anti-rotation setscrew
1-298-88	HPCS D/G storage tank	Remove manway
1-307-88	RCIC	Change out water leg pump orifice plate and relief valves 1E51-F017/18
1-335-88	"1B" D/G	Replace lug on engine control power
1-346-88	1E22-F011	HPCS to CY test, inspect rotation
1-347-88	HPCS	Uncouple motor/pump and drain system
1-355-88	HPCS pump	Surveillance LES-GM-106
1-367-88	B/C RHR	Surveillance LES-RH-101
1-371-88	1E12-F090B/92B	Locked valve administrative control

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-384-88	1VY02A	Hydrolaze area cooler
1-386-88	ACB 1433	Surveillance LES-GM-105
1-388-88	1E21-F016B/17B 1E51-F086	Surveillance LIS-RH-114
1-469-88	1E12-F017B	Surveillance LES-EQ-112
1-476-88	B/C RHR	Isolate and drain
1-477-88	B/C RHR	Isolate and drain
1-478-88	B/C RHR	Isolate and drain
1-485-88	1E12-F050B	Repair Sealtite
1-486-88	B/C RHR water leg pump 1E12-C003	Replace MCC contactor
1-487-88	1E12-F099B	Repair Sealtite
1-490-88	1E12-F003B	Determinate
1-491-88	1E12-F074B	LES-EQ-112
1-492-88	1E12-F024B	Determinate
1-520-88	1E22-F023	LES-GM-108 inspection
1-521-88	1E22-F010	LES-GM-108 inspection
1-522-88	1E22-F011	LES-GM-108 inspection
1-523-88	1E22-F012	LES-GM-108 inspection
1-524-88	1E22-F015	LES-GM-108 inspection
1-525-88	1E22-F001	LES-GM-108 inspection
1-526-88	1E22-F004	LES-GM-108 inspection
1-527-88	1E22-F003	LES-GM-108 inspection
1-528-88	"1B" D/G Petter Diesel A/C	LES-GM-108 inspection
1-529-88	HPCS Spool piece from CY	Connect temporary flush

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-532-88	"B" RHR pump 1E12-C002B	Chemically clean seal cooler
1-550-88	1E12-F003B	Remove and inspect limitorque motor
1-551-88	1E12-F024B	"1B" RHR pump
1-552-88	1E12-C002C	"C" RHR pump seal cooler
1-553-88	1E12-F048B	"B" RHR Hx 48B repair
1-554-88	1E12-F021	"C" RHR FFT repair
1-555-88	"B" RR pump drains	Install coupling
1-567-88	1E22-F023	Administrative control
1-570-88	1E51-F068 RCIC Turbine isolation	Administrative control-Floodup
1-575-88	1E12-F008	LES-EQ-112
1-593-88	1E12-F053A 1E12-F090A	Decon flush of 1RR0744 LLP-88-023

3. Off-Site Dose Calculation Manual

There were no changes to the OSDC manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to Radioactive Waste treatment systems during this reporting period.

5. Indications of Failed Fuel Elements

There were no indications of failed fuel elements during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MARCH, 1988

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0031r

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Steven J. Samolinski, telephone number (815)357-6761 extension 705.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

MARCH 1-31

March 1, 0001 hours	Unit Two entered March with the Reactor critical and the generator on-line at 1085 MWe.
March 7, 2100 hours	Load drop to 725 MWe to perform Heater Bay Surveillances and work.
March 8, 0630 hours	Begin ramping to 930 MWe.
March 9, 1739 hours	Reactor scram on APRM Hi-Hi.
March 18, 0307 hours	Reactor critical.
March 18, 1945 hours	Main Generator synchronized on to grid. Begin ramping to 440 MWe.
March 21, 1800 hours	Ramping to 1030 MWe.
March 30, 0001 hours	Load drop to 611 MWe for rod pattern adjustment.
March 30, 0530 hours	Begin ramping up to 1074 MWe.
March 31, 2400 hours	Reactor critical, generator on line and holding at 1074 MWe.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility license or Technical Specification.

Amendment Number: 36
Jet Pump Operability Surveillance.

Amendment Number: 37
Administrative Change to Include Nuclear Safety Review of all Reportable Occurrences.

2. Changes to procedures which are described in the Safety Analysis Report.

There were no changes made to procedures which are described in the Safety Analysis Report.

3. Tests and Experiments not described in the Safety Analysis Report.

There were no Tests or Experiments conducted which are not described in the Safety Analysis Report.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of Safety-Related Maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

5. Completed Safety Related Modifications.

The following table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L54478	FMCRD	Faulty position indicating probe	Improper position indication	Replaced probe
L75912	"C" outboard MSIV limit switch	Faulty limit switch	Prevented pickup of the K3F relay	Replaced limit switch
L75974	LPRM 40-17D	Bad LPRM card	LPRM Failed downscale	Replaced card
L76185	"B" RR inbd. isolation valve 2B33-F344B	Bad shell assembly coil and loose terminal connection	Valve closing spuriously	Replaced coil and tightened terminal connection
L76353	125 VDC DIV. I battery charger	Faulty inverter	Inconsistant charging amps	Repaired inverter
L76402	DIV. I Post LOCA Monitor 2PL76J	O_2 monitor out of calibration	Monitor reading high	Recalibrated
L76438	Alarm unit PS-2E32-N651E	Broken lead lug	Improper connection	Replaced lead lug
L76459	PCIS test switch	Bad contact block and faulty relay	Improper IN isolation	Replaced contact block and repaired faulty relay
L76528	HCU 46-19	Bad scram pilot valves	Improper HCU isolation	Replaced scram pilot valves
L76604	IRM "C" 2C51-K601C Detector OXY-VC165A	Bad MAV connection at IRM drawer	IRM spiking causing spurious half scrams	Replaced connector
L76605	IRM "D" 2C51-K601D	Faulty inhibit switch	Spurious half scrams	Replaced switch

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L76659	"B" RR HPU Inbd. Isolation Valves 2B33-F342B	Damaged lug inside valcour valve	Valves failed closed	Replaced lugs
L76667	"A" FW testable check valve	Seals failed on air cylinders	Compromised valve operation	Replaced actuator air cylinder.
L76668	"C" Inbd. MSIV 2B21-F022C	Inoperable Norgren valve	MSIV not closing properly when slow closing	Replaced valve
L75720	"C" Inbd. MSIV 2B21-F022C	Improperly adjusted limit switch	K3F relay would not reset	Readjusted limit switch
L28936	LPRM 32-57A	Improperly connected fuse	No input to process computer	Resoldered fuse to trace on LPRM card.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L54478	FMCRD	Faulty position indicating probe	Improper position indication	Replaced probe
L75912	"C" outboard MSIV limit switch	Faulty limit switch	Prevented pickup of the K3F relay	Replaced limit switch
L75974	LPRM 40-17D	Bad LPRM card	LPRM Failed downscale	Replaced card
L76185	"B" RR inbd. isolation valve 2B33-F344B	Bad shell assembly coil and loose terminal connection	Valve closing spuriously	Replaced coil and tightened terminal connection
L76353	125 VDC DIV. I battery charger	Faulty inverter	Inconsistant charging amps	Repaired inverter
L76402	DIV. I Post LOCA Monitor 2PL76J	O_2 monitor out of calibration	Monitor reading high	Recalibrated
L76438	Alarm unit PS-2E32-N651E	Broken lead lug	Improper connection	Replaced lead lug
L76459	PCIS test switch	Bad contact block and faulty relay	Improper IN isolation	Replaced contact block and repaired faulty relay
L76528	HCU 46-19	Bad scram pilot valves	Improper HCU isolation	Replaced scram pilot valves
L76604	IRM "C" 2C51-K601C Detector OXY-VC165A	Bad MAV connection at IRM drawer	IRM spiking causing spurious half scrams	Replaced connector
L76605	IRM "D" 2C51-K601D	Faulty inhibit switch	Spurious half scrams	Replaced switch

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER: A brief Synopsis of Incorporated Modification Objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

UNIT TWO

There were no safety related modifications completed on Unit Two during the reporting period of March 1 through March 31, 1988.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, February 1, through February 29, 1988. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
88-001-00	3/2/88	2B D/G Circulating Water Pump Failure to Auto Start.
88-002-00	3/3/88	30 Day - Group 10 Div. II Isolation.
88-003-00	3/9/88	Scram 1 - Reactor Scram on High APRM Flux.
88-004-00	3/21/88	30 Day - Missed Surveillance on LOS-MS-M1.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE April 10, 1988
COMPLETED BY Steven J. Samolinski
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: March, 1988 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY): None

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	542.5	1982.5	18767.3
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	29.83
7.	HOURS GENERATOR ON LINE	525.9	1965.9	18437.0
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	1419096	5840640	53635696
10.	GROSS ELEC. ENERGY GENERATED (MWH)	470842	1953935	17762146
11.	NET ELEC. ENERGY GENERATED (MWH)	450050	1883362	16965975
12.	REACTOR SERVICE FACTOR	72.9%	90.8%	62.0%
13.	REACTOR AVAILABILITY FACTOR	72.9%	90.8%	62.1%
14.	UNIT SERVICE FACTOR	70.7%	90.0%	60.9%
15.	UNIT AVAILABILITY FACTOR	70.7%	90.0%	60.9%
16.	UNIT CAPACITY FACTOR (USING MDC)	58.4%	83.2%	54.1%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	56.1%	80.0%	52.0%
18.	UNIT FORCED OUTAGE RATE	29.3%	10.0%	19.6%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP. N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: April 10, 1988
COMPLETED BY: Steven J. Samolinski
TELEPHONE: (815) 357-6761
MONTH: MARCH, 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	1049	17.	- 18
2.	1043	18.	13
3.	1044	19.	385
4.	1045	20.	398
5.	1040	21.	475
6.	1037	22.	904
7.	988	23.	932
8.	810	24.	922
9.	637	25.	897
10.	- 7	26.	864
11.	- 7	27.	884
12.	- 14	28.	874
13.	- 18	29.	921
14.	- 19	30.	773
15.	- 19	31.	939
16.	- 18		

DOCUMENT ID 0036r/0031r

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE April 10, 1988
COMPLETED BY S. Samolinski
TELEPHONE (815)357-6761

REPORT MONTH MARCH, 1988

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
5	3/7/88	S	0.0	B	5	Heater Bay work and surveillances.
6	3/9/88	F	218.10	A	3	Reactor scrammed on APRM Hi-Hi and loss of both RR pumps.
7	3/30/88	S	0.0	B	5	Rod pattern adjustment.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety Relief Valves actuated during the reporting period of March 1 through March 31, 1988.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-131-88	"2B" D/G	Clean K18 relay contacts
2-145-88	2DG08CA "2A" D/G "A" A/C	Head relief lifts
2-171-88	"B/C" RHR Water leg pump	Replace EQ contactor
2-176-88	"C" RHR pump	Motor EQ terminations
2-196-88	RCIC Barometric condensor vacuum pump	Inspect motor
2-209-88	RCIC	Lubrication
2-230-88	2E12-F004B	Repair valve actuator
2-237-88	"2A" D/G	Lubrication
2-242-88	"2B" D/G air compressor	Change high pressure valve assembly
2-249-88	2E12-F049A/B	Prevent use
2-250-88	2E12-D313	Replace Sightglass
2-252-88	2E12-F049A/B	Repair 2E12-D313

3. Off-Site Dose Calculation Manual

There were no changes to the OSDC manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to Radioactive Waste treatment systems during this reporting period.

5. Indications of Failed Fuel Elements.

Off Gas Levels indicate that there is one pinhole fuel element failure in the reactor. This does not represent a change from the previous reporting period.

There were no indications of failed fuel elements during this reporting period.

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April 10, 1988

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period March 1, 1988 through March 31, 1988.

Very truly yours,

G. J. Diederich
Station Manager
LaSalle County Station

GJD/SJS/jdp

Enclosure

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