Omaha Public Power District 1623 Harney Omaha, Nebraska 68102 402/536 4000

January 28, 1988 LIC-88-050

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

References: 1. Docket No. 50-285 2. Letter from NRC (W. A. Paulson) to OPPD (R. L. Andrews) dated January 20, 1987

Gentlemen:

SUBJECT: Amendment 102 - Correction

An error has been found in Amendment 102, page 2-9. The figure u is missing from the second paragraph of "Basis" (see attached). When the original application was sent in, this item was not a part of the changes made, but must have been inadvertently left out when retyped for issuance.

The error is only in the "Basis", therefore no application is required. If you should have any questions concerning this matter please contact us.

Sincerely,

Mandruss

R. L. Andrews Division Manager Nuclear Production

RLA/me

Attachment

cc: LeBoeuf, Lamb, Leiby & MacRae 1333 New Hampshire Ave., N.W. Washington, DC 20036

> R. D. Martin, NRC Regional Administrator A. Bournia, NRC Project Manager P. H. Harrell, NRC Senior Resident Inspector

8802020154 880128 PDR ADOCK 05000285 P PDR

2.0 LIMITING CONDITIONS FOR OPERATION

2.1 Reactor Coolant System (Continued)

2.1.3 Reactor Coolant Radioactivity (Continued)

e. Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level.

Basis

The limitations on the radioactivity of the reactor coolant ensure that the resulting 2-hour doses at the site boundary will be well within the limits of 10 CFR Part 100 following a steam generator tube rupture accident in conjunction with an assumed stead/ state primary-to-secondary steam generator leakage rate of 1.0 GPM and a concurrent loss of offsite power.

Permitting operation to continue for limited time periods with the reactor coolant's radioactivity levels >1.0 μ Ci/gm DOSE EQUIVALENT I-131, but < 60 Ci/gm, accommodates possible iodine spiking phenomenon which may occur following changes in thermal power.

Reducing Tavg to < 536°F prevents the release of radioactivity should a steam generator tube rupture, since the saturation pressure of the reactor coolant is below the lift pressure of the atmospheric steam relief valves. The surveillance requirements provide adequate assurance that excessive radioactivity levels in the reactor coolant will be detected in sufficient time to take appropriate corrective action(s).

References

USAR, Section 11.11.3

USAR, Section 14.14

Amendment No. 60,67, 102

2-9