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April 4, 1988

Docket No. 50-423 B12871

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

References:

- E. J. Mroczka letter to U.S. Nuclear Regulatory Commission, Physics Methodology for PWR Reload Design, dated November 13, 1987.
- (2) E. J. Mroczka letter to U.S. Nuclear Regulatory Commission, Physics Methodology for PWR Reload Design, dated March 2, 1958.

Gentlemen:

Millstone Nuclear Power Station, Unit No. 3
Physics Methodology for PWR Reload Design (TAC No. 66693)

In Reference (1), Northeast Utilities Service Company (NUSCO), on behalf of Northeast Nuclear Energy Company (NNECO), submitted an addendum to a topical report (NUSCO-152) demonstrating NUSCO's ability to perform pressurized water reactor reload physics design using Westinghouse physics methodology. This addendum summarizes the comparison of zero power physics test data and at power measurements to predictions for Cycle 1 of Millstone Unit No. 3. The following information is being sent in response to an NRC request, made during a telephone conference between the NRC and the NNECO representative on March 17, 1988.

NUSCO will perform a periodic reevaluation of its model validity by a comparison of predicted to measured results from the Low Power Physics Test Program for each reload cycle. The test program and the test review criteria to be used for design review will be consistent with those proposed in ANSI/ANS 19.6.1-1985, "American National Standard Reload Startup Physics Tests for Pressurized Water Reactors."

The design review criteria is a guide to possible measurement or design errors. Exceeding these criteria will initiate a review of the measurement data as well as the design calculations and models used to provide predicted results. Any errors discovered will be accounted for and corrected, as appropriate. A review of other design data that may be adversely affected due to measurement discrepancies will also be performed.

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Safety review criteria are set separately from the design review criteria and are based upon safety analysis calculations and/or Plant Technical Specifications.

We believe the information provided above, coupled with the information provided in References (1) and (2), provides a complete basis for approval of the addendum to NUSCO-152. Of course, should the Staff have any additional questions, we are available to discuss the Staff's concerns.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

Senior Vice President

cc: M. T. Russell, Region I Administrator

R. L. Ferguson, NRC Project Manager, Millstone Unit No. 3

W. J. Raymond, Senior Resident Inspector, Millstone Unit Nos. 1, 2, and 3