

Northeast
Nuclear Energy

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Millstone Nuclear Power Station
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The Northeast Utilities System

Docket No. 50-336
B16519


JUN 13 1997

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 2
Facility Operating License No. DPR-65
Monthly Operating Report

In accordance with the reporting requirements of Technical Specification Section 6.9.1.7 for Millstone Unit No. 2, enclosed in Attachment 1, is the monthly operation report for the month of May 1997.

NORTHEAST NUCLEAR ENERGY COMPANY

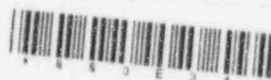


J. A. Price
Director, Millstone Unit No. 2

cc: H. J. Miller, Region I Administrator
D. G. McDonald, Jr., NRC Project Manager, Millstone Unit No. 2
D. P. Beaulieu, Senior Resident Inspector, Millstone Unit No. 2
W. D. Travers, PhD, Director, Special Projects Office

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Attachment 1

Millstone Nuclear Power Station, Unit No. 2

Facility Operating License No. DPR-65
Monthly Operating Report

June 1997

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-336
 UNIT Millstone Unit 2
 DATE 6/6/97
 COMPLETED BY S. Stark
 TELEPHONE (860) 447-1791
 EXT 4419

MONTH: MAY 1997

DAY	AVG. DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVG. DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

UNIT NAME Millstone Unit 2
 DATE 06/06/97
 COMPLETED BY S. Stark
 TELEPHONE (860) 447-1791
 EXT 4419

OPERATING STATUS

1. Docket Number 50-336
 2. Reporting Period May 1997
 3. Utility Contact S. Stark
 4. Licensed Thermal Power (MWt): 2700
 5. Nameplate Rating (Gross MWe): 909
 6. Design Electrical Rating (Net MWe): 870
 7. Maximum Dependable Capacity (Gross MWe): 901.63
 8. Maximum Dependable Capacity(Net MWe): 870.63
 9. If Changes Occur in Capacity Ratings (Items Number 4 Through 8) Since Last Report, Give Reasons:
 N/A

Notes: Items 22 and 23 cumulative are weighted averages. Unit operated at 2560 MWTH prior to its uprating to its current 2700 MWTH power level.

10. Power Level To Which Restricted, If any (Net MWe): 0
 11. Reasons For Restrictions, If Any: NRC Category III Facility; NRC Confirmatory Order requiring implementation of an independent corrective action verification program; NRC order requiring a third-party review of the employee concerns program at Millstone 2; design basis verification response pursuant to 10CFR50.54(f).

	This Month	Yr.-To-Date	Cumulative
12. Hours In Reporting Period	744.0	3623.0	187871.0
13. Number Of Hours Reactor Was Critical	0.0	0.0	121911.7
14. Reactor Reserve Shutdown Hours	0.0	0.0	2205.5
15. Hours Generator On-Line	0.0	0.0	116611.9
16. Unit Reserve Shutdown Hours	0.0	0.0	468.2
17. Gross Thermal Energy Generated (MWH)	0.0	0.0	300862506.4
18. Gross Electrical Energy Generated (MWH)	0.0	0.0	98709460.0
19. Net Electrical Energy Generated (MWH)	-2050.7	-10929.9	94630253.5
20. Unit Service Factor	0.0	0.0	62.1
21. Unit Availability Factor	0.0	0.0	62.3
22. Unit Capacity Factor (Using MDC Net)	0.0	0.0	58.8
23. Unit Capacity Factor (Using DER Net)	0.0	0.0	58.0
24. Unit Forced Outage Rate	100.0	100.0	21.3
25. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Shutdown at the time of this report			

26. If Unit Shutdown At End Of Report Period, Estimated Date of Startup: To be determined
 27. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-336
 UNIT NAME Millstone Unit 2
 DATE 06/02/97
 COMPLETED BY S. Doboie
 TELEPHONE (860) 447-1791
 EXT 4419

REPORT MONTH: May 1997

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	License Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
97-01	03/07/96	S/F	744	B/D	4	N/A	N/A	N/A	<p>Scheduled: Continued mid cycle surveillance testing from previous year.</p> <p>Forced: Continued from previous year. NRC Category III facility; NRC Confirmatory Order requiring independent corrective action verification; NRC order requiring third party review of Millstone Station employee concerns program; design basis verification for response to NRC pursuant to 10CRF50.54(f).</p>

¹F: Forced
 S: Scheduled

²Reason
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & License Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

³Method
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Continued from Previous Month
 5 - Power Reduction (Duration = 0)
 6 - Other (Explain)

⁴IEEE Standard 805-1984, "Recommended Practices for System Identification in Nuclear Power Plants and Related Facilities"

⁵IEEE Standard 803A-1983, "Recommended Practices for Unique identification in Power Plants and Related Facilities - Component Function Identifiers"

REFUELING INFORMATION REQUEST

1. Name of the facility: Millstone Unit 2
2. Scheduled date for next refueling outage: First Quarter of 1999
3. Scheduled date for restart following refueling: Last Quarter of 1997 (Note - The current shutdown is not a refueling outage. This date represents the expected startup date from the current shutdown.)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
Yes. There are 15 Technical Specification Change Requests which will be submitted to the NRC prior to startup. These include the following: 1) Modify applicability and actions associated with Steam Generator blowdown monitor and modify applicability for Condensate Polishing Facility Waste Neutralization Sump. 2) Modify the minimum instrument accuracy for Meteorological Tower instrumentation. 3) Eliminate the reference to the measurement location for the ultimate heat sink. 4) Eliminate the requirement to enter LCO 3.0.3 when the enclosure building is breached, due to making both trains of enclosure building ventilation system inoperable (unable to create a vacuum within specified time period). 5) Redefine containment integrity to permit operator action during periods when containment isolation valves may be opened under administrative controls. Also relocates the list of containment isolation valves from Technical Specifications to Technical Requirements Manual. 6) Modify the bases for specification to resolve the emergency diesel generator fuel oil storage capacity issues. 7) Add 2 second time delay to ESFAS Table 3.3-4 8a to be consistent with 8b. 8) Delete Technical Specification 3/4.9.14 and Table 3.9-1. Modify LCO 3.9.15 to require two EBFS trains to be operable and add the requirement to have at least one train in operation whenever fuel movement or crane operation is in progress. Add a requirement regarding spent fuel pool ventilation integrity, change the associated action requirements, change the surveillance requirements, and the bases. This change removes the requirement for AEAS to be operating prior to fuel movement. 9) Modify action statement of LCO 3.4.1.3 for no shutdown cooling loops operable. 10) Revise several surveillance requirements regarding Control Room Ventilation System. 11) Certain RPS and ESAS trips did not consider the uncertainty associated with a Harsh Environment. Some RPS and ESAS setpoints were based on 24 months instead of 18 months. 12) Verbatim compliance issues. ECCS throttle valves. AFW surv. 13) Correct RCS Pressure/Temperature curves due to errors.

14) RPS and ESAS trip action statements allow indefinite bypass of an inoperable channel. This will be changed to allow only 48 hours before placing the channel in the tripped condition. 15) Modify Table 3.7-1 to correct errors. Reactor trip setpoints for steam generator safety valves in Table 3.7-1 are incorrectly based on 107% power instead of 106.6%.

5. Scheduled date(s) for submitting licensing action and supporting information:
July 30, 1997

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
Small Break LOCA analysis changes, Large Break LOCA analysis changes, and Uncontrolled Rod Withdrawal analysis changes.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

In Core: (a) 0 In Spent Fuel Pool: (b) 1085

NOTE: These numbers represent the total Fuel Assemblies and Consolidated Fuel Storage Boxes (3 total containing the fuel rods from 6 fuel assemblies) in these two (2) Item Control Areas.

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:
Present storage capacity: 1306 storage locations

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming present license capacity:

2001, Spent Fuel Pool Full, Core offload capacity is reached.

2005, Core Full, Spent Fuel Pool Full.