NRC FORM 366 (4-95) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 04/30/98

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Millstone Nuclear Power Station Unit 3	05000423	1 of 3

TITLE (4)

Calibration of Pumped Capacity Monitoring System Timers Not In Accordance With 18 Month Surveillance Requirement of the Technical Specifications

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER		REVISION NUMBER		MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	16	97	97	--	032	--	00	06	13	97	FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)			
POWER LEVEL (10)	000	20.2201(b)	20.2203(a)(2)(v)	[X] 50.73(a)(2)(i)	50.73(a)(2)(viii)
		20.2203(a)(1)	20.2203(a)(3)(i)	50.73(a)(2)(ii)	50.73(a)(2)(x)
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)	50.73(a)(2)(iii)	73.71
		20.2203(a)(2)(ii)	20.2203(a)(4)	50.73(a)(2)(iv)	OTHER
		20.2203(a)(2)(iii)	50.36(c)(1)	50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A
		20.2203(a)(2)(iv)	50.36(c)(2)	50.73(a)(2)(vii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
David A. Smith, MP3 Nuclear Licensing Manager	(860)437-5840

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
[ ] YES (If yes, complete EXPECTED SUBMISSION DATE).	[X] NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On May 16, 1997, with the unit in Mode 5, it was identified that the run timer-relays (62A and 62B) associated with the unidentified leakage sump pump (3DAS-P10), had not been tested at the proper frequency. These relays are part of the Containment Drain Sump Level and Pumped Capacity Monitoring System listed in Technical Specification (TS) 3.4.6.1, "Reactor Coolant System Leakage, Leakage Detection Systems." TS Surveillance Requirement (SR) 4.4.6.1.b states that the "Containment Drain Sump Level and Pumped Capacity Monitoring System - be tested by performance of a CHANNEL CALIBRATION at least once per 18 months. Contrary to this, the unidentified leakage sump pump run timer-relays have been calibrated on a 3 year frequency as part of the Preventative Maintenance program since initial plant startup. Therefore, because calibration of the unidentified leakage sump pump timer-relays was not performed every 18 months as required by SR 4.4.6.1.b, the CHANNEL CALIBRATION requirements were not met. This condition is being reported pursuant to 10 CFR 50.73(a)(2)(i)(B) as an operation or condition prohibited by the unit's Technical Specifications.

The cause of this historical event was inadequate review and coordination between the instrumentation and controls department and, electrical maintenance organization during initial surveillance development.

This Technical Specification is applicable in Modes 1-4, and because the plant was in Mode 5, no immediate corrective actions were necessary. The Containment Drain Sump Level and Pumped Capacity Monitoring System surveillance will be revised to include the timer-relays.

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TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

I. Description of Event

On May 16, 1997, with the unit in Mode 5, it was identified that the run timer-relays (62A and 62B) associated with the unidentified leakage sump pump (3DAS-P10), had not been tested at the proper frequency. These relays are part of the Containment Drain Sump Level and Pumped Capacity Monitoring System listed in Technical Specification (TS) 3.4.6.1, "Reactor Coolant System [RCS] Leakage, Leakage Detection Systems." TS Surveillance Requirement (SR) 4.4.6.1.b states that the "Containment Drain Sump Level and Pumped Capacity Monitoring System - be tested by performance of a CHANNEL CALIBRATION at least once per 18 months. TS Section 1.5, states within the definition of a CHANNEL CALIBRATION, that; "... The CHANNEL CALIBRATION shall encompass the entire channel including the sensors and alarm, interlock and/or trip functions and may be performed by any series of sequential, overlapping, or total channel steps such that the entire channel is calibrated." Contrary to this, the unidentified leakage sump pump run timer-relays have been calibrated on a 3 year frequency as part of the Preventative Maintenance (PM) program since initial plant startup. Therefore, because calibration of the unidentified leakage sump pump timer-relays was not performed every 18 months as required by SR 4.4.6.1.b, the CHANNEL CALIBRATION requirements were not met. This condition is being reported pursuant to 10 CFR 50.73(a)(2)(i)(B) as an operation or condition prohibited by the unit's Technical Specifications.

This Technical Specification is applicable in Modes 1-4, and because the plant was in Mode 5, no immediate corrective actions were necessary.

II. Cause of Event

The cause of this historical event was inadequate review and coordination between the instrumentation and controls department and, electrical maintenance organization during initial surveillance procedure development. This is a historical event, identified as part of the Configuration Management Program review process, needed to support Millstone Unit 3 restart.

III. Analysis of Event

Leakage within containment from the Reactor coolant pressure boundary (RCPB) [other than normal or identified leakage collected in the containment drains transfer tank or in the pressurizer relief tank], is collected in the unidentified leakage sump. Sump level change and sump pump run time are utilized to determine the rate of flow of unidentified leakage into the sump. As described in the Final Safety Analysis Report (FSAR), these detection methods are capable of detecting a 1 gallon-per-minute (gpm) change in leakage rate into the sump within one hour. The run-timer relays provide an alarm to alert the operators that there may be an unidentified leak in the RCPB. One alarm is based on the unidentified leakage sump pump running too long. The other alarm is based on the sump refilling too quickly.

The Containment Drain Sump Level or Pumped Capacity Monitoring System, while inoperable in accordance with TS surveillance frequency requirements, was still available, and capable of detecting leakage rates in the vicinity of 1 gpm, but may have been unable to meet the detection requirements specified in the FSAR. Because the system could still detect gross leakage, calibrating the timer-relays on a 3 year cycle versus the every 18 months TS cycle had minimal safety significance. Additionally, the plant computer independently monitors the unidentified leakage sump level and provides an alarm if leakage to the unidentified leakage sump exceeds 1 gpm within one hour.

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TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

IV. Corrective Action

The following corrective actions will be taken:

1. The Containment Drain Sump Level and Pumped Capacity Monitoring System surveillance will be revised to include the timer-relays by August 13, 1997.

2. The revised Containment Drain Sump Level and Pumped Capacity Monitoring System surveillance will be performed to verify Technical Specification compliance by September 30, 1997.

V. Additional Information

None

Similar Events

Five LERs discussing missed or incomplete surveillances are identified below. Various elements of the Configuration Management Program are being conducted to detect design and licensing basis problems, which includes the Technical Specifications. The LERs are:

LER 96-002-00	Inadequate Surveillance for Determining Shutdown Margin When Unisolating a Reactor Coolant Loop, Due to Procedure Inadequacy
LER 96-021-00	Components Not Included in the In-Service Test Program as a Result of Programmatic Deficiencies
LER 96-034-00	RHR Pump Suction Relief Valve Setpoint Not In Accordance With Technical Specifications
LER 97-006-00	Residual Heat Removal Suction Isolation Valves Open But Not Under Administrative Control as Required in Mode 4 by Surveillance Requirement 4.6.1.1.a
LER 97-031-00	RHR Valve Low Pressure Open Permissive Bistable Setting Set Non-Conservatively

Manufacturer Data

EIIS System Code

Leak Monitoring System.....IJ
Reactor Coolant System.....AB

EIIS Component Code

Relay.....RLY