

ATTACHMENT 1

PROPOSED TECHNICAL SPECIFICATIONS CHANGES

Virginia Electric and Power Company

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Containment Pressure	2	1
2. Reactor Coolant Inlet Temperature- $T_{hot}$ (wide range)	2	1
3. Reactor Coolant Inlet Temperature- $T_{hot}$ (wide range)	2	1
4. Reactor Coolant Pressure-Wide Range	1	1
5. Pressurizer Water Level	1	1
6. Steam Line Pressure	2/steam generator	1/steam generator
7. Steam Generator Water Level-Narrow Range	2/steam generator	1/steam generator
8. Refueling Water Storage Tank Water Level	1	1
9. Boric Acid Tank Solution Level	1	1
10. Auxiliary Feedwater Flow Rate	1/steam generator	1/steam generator
11. Reactor Coolant System Subcooling Margin Monitor	2	1
12. PORV Position Indicator	2/valve	1/valve
13. PORV Block Valve Position Indicator	1/valve	1/valve
14. Safety Valve Position Indicator	1/valve	1/valve
15. Reactor Vessel Coolant Level Monitor	2	1
16. Containment Water Level (narrow range)	2	1
17. Containment Water Level (wide range)	2	1
18. In Core Thermocouples	4/core quadrant	2/core quadrant

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Containment Pressure	M	R
2. Reactor Coolant Inlet Temperature-T <sub>hot</sub> (wide range)	M	R
3. Reactor Coolant Inlet Temperature-T <sub>hot</sub> (wide range)	M	R
4. Reactor Coolant Pressure-Wide Range	M	R
5. Pressurizer Water Level	M	R
6. Steam Line Pressure	M	R
7. Steam Generator Water Level-Narrow Range	M	R
8. Refueling Water Storage Tank Water Level	M	R
9. Boric Acid Tank Solution Level	M	R
10. Auxiliary Feedwater Flow Rate	M	R
11. Reactor Coolant System Subcooling Margin Monitor	M	R
12. PORV Position Indicator	M	R
13. PORV Block Valve Position Indicator	M	R
14. Safety Valve Position Indicator	M	R
15. Reactor Vessel Coolant Level Monitor	M	R
16. Containment Water Level (narrow range)	M	R
17. Containment Water Level (wide range)	M	R
18. In Core Thermocouples	M	R

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Containment Pressure	2	1
2. Reactor Coolant Inlet Temperature-T <sub>hot</sub> (wide range)	2	1
3. Reactor Coolant Inlet Temperature-T <sub>hot</sub> (wide range)	2	1
4. Reactor Coolant Pressure-Wide Range	1	1
5. Pressurizer Water Level	1	1
6. Steam Line Pressure	2/steam generator	1/steam generator
7. Steam Generator Water Level-Narrow Range	2/steam generator	1/steam generator
8. Refueling Water Storage Tank Water Level	1	1
9. Boric Acid Tank Solution Level	1	1
10. Auxiliary Feedwater Flow Rate	1/steam generator	1/steam generator
11. Reactor Coolant System Subcooling Margin Monitor	2	1
12. PORV Position Indicator	2/valve	1/valve
13. PORV Block Valve Position Indicator	1/valve	1/valve
14. Safety Valve Position Indicator	1/valve	1/valve
15. Reactor Vessel Coolant Level Monitor	2	1
16. Containment Water Level (narrow range)	2	1
17. Containment Water Level (wide range)	2	1
18. In Core Thermocouples	4/core quadrant	2/core quadrant

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Containment Pressure	M	R
2. Reactor Coolant Inlet Temperature-T <sub>hot</sub> (wide range)	M	R
3. Reactor Coolant Inlet Temperature-T <sub>hot</sub> (wide range)	M	R
4. Reactor Coolant Pressure-Wide Range	M	R
5. Pressurizer Water Level	M	R
6. Steam Line Pressure	M	R
7. Steam Generator Water Level-Narrow Range	M	R
8. Refueling Water Storage Tank Water Level	M	R
9. Boric Acid Tank Solution Level	M	R
10. Auxiliary Feedwater Flow Rate	M	R
11. Reactor Coolant System Subcooling Margin Monitor	M	R
12. PORV Position Indicator	M	R
13. PORV Block Valve Position Indicator	M	R
14. Safety Valve Position Indicator	M	R
15. Reactor Vessel Coolant Level Monitor	M	R
16. Containment Water Level (narrow range)	M	R
17. Containment Water Level (wide range)	M	R
18. In Core Thermocouples	M	R

ATTACHMENT 2

SAFETY EVALUATION

Virginia Electric and Power Company

## DISCUSSION

Generic Letter No. 83-37 "NUREG-0737 TECHNICAL SPECIFICATIONS," requires that Technical Specifications provide assurance that facility operation is maintained within acceptable limits for each facility in accordance with NUREG-0737.

This change will add In Core Thermocouples to Technical Specification Table 3.3-10, ACCIDENT MONITORING INSTRUMENTATION and require 4 thermocouples per core quadrant for OPE<sup>R</sup>ABILITY determination while allowing 2 thermocouples per quadrant as the minimum number of operable channels for operation. Also the In Core Thermocouples will be added to Tech. Spec. Table 4.3-7, ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS, requiring a "Channel Check" on a monthly basis and a "Channel Calibration" each refueling.

These changes conform to the sample provided in Enclosure 3 to Generic Letter 83-37, pages 15 and 16.

### BASIS FOR NO SIGNIFICANT HAZARDS DETERMINATION

We have reviewed this change to Technical Specifications and have found that it is similar to 48FR14870, "Examples of Amendments that are not Likely to Involve Significant Hazards Considerations are Listed Below," Example (ii) "A change that constitutes an additional limitation, restriction, or control not presently included in the technical specifications: for example, a more stringent surveillance requirement," and Example (vii) "A change to make a license conform to changes in regulations, where the license change results in very minor changes to facility operations clearly in keeping with the regulations."

The proposed change does not involve a significant hazards consideration because operation of North Anna Units 1 and 2 in accordance with this change would not:

- (1) involve a significant increase in the probability or consequence of an accident previously evaluated, because the change will require additional controls and surveillances in the operation of the plant and is in accordance with Generic Letter 83-37 and NUREG-0737. The operability requirements and the surveillance requirements for the accident monitoring instrumentation have not been decreased by this change.
- (2) create the possibility of a new or different kind of accident from any accident previously identified, because the change will require additional controls and surveillances in the operation of the plant and is in accordance with Generic Letter 83-37 and NUREG-0737. The proposed change does not involve any alterations to plant equipment or procedures which would introduce any new or unique operational modes or accident precursors.
- (3) involve a significant reduction in a margin of safety, because the change will require additional controls and surveillances in the operation of the plant and is in accordance with Generic Letter 83-37 and NUREG-0737. The results of the accident analyses performed in the UFSAR will continue to bound operation.

Therefore, pursuant to 10 CFR 50.92, based on the above consideration, it has been determined that this change does not involve a significant safety hazards consideration.