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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

January 15, 1988

Docket No. 50-482

Mr. Bart D. Withers President and Chief Executive Officer Wolf Creek Nuclear Operating Corporation Post Office Box 411 Burlington, Kansas 66839

Dear Mr. Withers:

SUBJECT: FIRST TEN YEAR INTERVAL INSERVICE TESTING PROGRAM AND REQUESTS FOR RELIEF FROM CERTAIN REQUIREMENTS FOR WOLF CREEK GENERATING STATION (TAC NO. 57359)

By letter dated September 5 and September 18, 1981 and supplements thereto dated July 31, 1984, February 25, 1985, July 29, 1985, June 10, 1986, March 2, 1987, April 24, 1987 and September 11, 1987, Kansas Gas and Electric Company (the licensee, presently Wolf Creek Nuclear Operating Corporation) submitted the Wolf Creek Generating Station first 10-year Interval Inservice Testing (IST) Program Plan to the NRC for review and evaluation of its compliance with the requirements of Section XI of the ASME Code, 10 CFR 50.55 and additional information related to requests for relief from certain code requirements determined to be impractical to perform on the Wolf Creek Generating Station. The NRC staff with technical assistance from the Idaho National Engineering Laboratory (INEL), has reviewed and evaluated the program and additional information provided by the licensee.

Certain relief requests have been denied where proposed alternative testing is unacceptable or where adequate basis for the request has not been provided. Also, certain omissions and inconsistencies, that were identified during the review, are identified in Appendix C of the TER. These items should be addressed within 90 days by either correcting the IST program or submitting additional relief requests, if appropriate.

Based upon the NRC staff's review of your IST program and relief requests we conclude that the program as evaluated and modified by this SER will provide reasonable assurance of the operational readiness of safety-related pumps and valves to perform their safety-related functions and the program for Wolf Creek Generating Station through the submittal of March 2, 1987 is acceptable for implementation provided that the items noted above are corrected promptly. Relief requests contained in any subsequent revisions may not be implemented without prior approval by NRC.

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Mr. Bart D. Withers

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The staff's safety evaluation and the INEL Technical Evaluation Report are enclosed. A summary of the relief requests evaluated and the determinations made by the staff is shown in Table 1 of the enclosed safety evaluation.

Sincerely,

Jose A. Calvo, Director Project Directorate - IV Division of Reactor Projects - III, IV, V and Special Projects Office of Nuclear Reactor Regulation

Enclosures: As stated

cc w/enclosures: See next page

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cc:

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