

ENCLOSURE 1

NOTICE OF VIOLATION

Duke Power Company
Catawba 1 and 2

Docket Nos. 50-413, 50-414
License Nos. NPF-35, NPF-52

During the Nuclear Regulatory Commission (NRC) inspection conducted on January 26, 1988 through February 25, 1988, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions", 10 CFR Part 2, Appendix C (1987), the violations are listed below:

A. Technical Specification 6.8.1 requires that written procedures shall be established, implemented, and maintained covering the applicable procedures recommended in Appendix A to Regulatory Guide 1.33, Revision 2.

1. OP/1/A/6100/01, Controlling Procedure for Unit Startup, step 2.60 cautions the operator not to exceed 1955 psig reactor coolant system pressure prior to steam generator pressure being greater than or equal to 725 psig as this will cause a safety injection on low steam pressure.

Contrary to the above, on January 23, 1988, reactor coolant system pressure was allowed to exceed 1955 psig prior to steam generator pressure being greater than or equal to 725 psig. This resulted in a safety injection on Unit 1.

2. Catawba Nuclear Station Directive 3.1.1 Safety Tags and Delineation tags sections 5.3.5 and 8.4.4 requires the person designated to remove a safety tag to receive the completed tag stub from the recalling authority, obtain recall approval from the recall authority, take the stub to the equipment and verify the tag number and stub number agree prior to removing the tag.

Contrary to the above, on February 9, 1988, an equipment operator designated to remove safety tag #410 of tagout number 28-223 on the SSPS Output Relay Mode Selector Switch, removed the tag and repositioned the switch without having received the completed tag stubs and without having obtained recall approval from the recall authority (Assistant Shift Supervisor). This resulted in an inadvertent safety injection on Unit 2.

These examples in the aggregate are a Severity Level IV Violation (Supplement I).

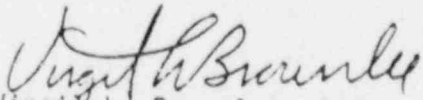
- B. 10 CFR 50, Appendix B, Criterion XVI as implemented by Quality Assurance Program (Duke 1-A, Amendment II) Section 17.2.16 requires that measures be established to assure that the cause of significant conditions adverse to quality be determined and corrective action taken to preclude repetition and items of the same type be evaluated to determine whether or not they can be expected to continue to function.

Contrary to the above, corrective action measures to preclude repetition of a significant condition adverse to quality were inadequate in that during the implementation of Temporary Station Modification Work Request 5831 IAE and Exempt Change CE-0840, to correct faulty motor mounting hardware for 2ND-36B, the licensee failed to properly evaluate applicability of the modification to 2ND-2A. Being that the modification was not applied to 2ND-2A, this contributed to the motor of 2ND-2A ejecting from the actuator when operated on February 1, 1988.

This is a Severity Level V Violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Duke Power Company is hereby required to submit a written statement or explanation to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555, with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector within thirty (30) days of the date of the letter transmitting this Notice. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) admission or denial of the violation, (2) the reason for the violation if admitted, (3) the corrective steps which have been taken and the results achieved, (4) the corrective steps which will be taken to avoid further violations, and (5) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending the response time. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked or why such other action as may be proper should not be taken.

FOR THE NUCLEAR REGULATORY COMMISSION


Virgil L. Brownlee, Chief
Reactor Projects Branch 3
Division of Reactor Projects

Dated at Atlanta, Georgia
this 10th day of March 1988