JAN 1 9 1988

Docket No. 50-219

× ..

DISTRIBUTION Docket File

NRC & Local PDRs CGC-Bethesda PDI-4 Reading S. Varca B. Boger S. Norris

A. Dromerick E. Jordan J. Partlow ACRS (10) Gray File

Mr. P. B. Fielder Vice President and Director Oyster Creek Nuclear Generating Station Post Office Box 388 Forked River, New Jersey 08731

Dear Mr. Fielder:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON ATWS REVIEW -OYSTER CREEK NUCLEAR GENERATING STATION (TAC NO. 66126)

The staff has reviewed the information provided by GPU Nuclear Corporation in letters dated September 3, and December 30, 1987 concerning implementation of the ATWS rule requirements at the Cyster Creek Nuclear Generating Station. Based on our review of this information, we find that additional information is required. The information needed by the staff to complete the evaluation is defined in the enclosure. You are requested to respond to this request for additional information by February 10, 1988 in order to meet the scheduled completion date.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 97-511.

Sincerely.

"PRINCIPAL STREET BY."

Alexander W. Dromerick, Project Manager Project Directorate I-4 Division of Reactor Projects I/II Division of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

LA: PDI-4 SNOTTIS ick:bd 01/19/88 188

K /88

8801260520 880119 ADOCK 05000219 PDR PDR

Mr. P. B. Fiedler Oyster Creek Nuclear Generating Station

Ernest L. Blake, Jr. Shaw, Pittman, Potts and Trowbridge 2300 N Street, NW Washington, D.C. 20037

J.B. Liberman, Esquire Bishop, Liberman, Cook, et al. 1155 Avenue of the Americas New York, New York 10036

Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, Pennsylvania 19406

BWR Licensing Manager GPU Nuclear Corporation 1 Upper Pond Road Parsippany, New Jersey 07054

Deputy Attorney General State of New Jarsey Department of Law and Public Safety 36 West State Street - CN 112 Trenton, New Jersey 08625

## Mayor Lacey Township 818 West Lacey Road Forked River, New Jersey 08731

Licensing Manager Oyster Creek Nuclear Generating Station Mail Stop: Site Emergency Bldg. P. O. Box 388 Forked River, New Jersey 08731 Oyster Creek Nuclear Generating Station

Resident Inspector c/o U.S. NRC Post Office Box 445 Forked River, New Jersey 08731

Commissioner New Jersey Department of Energy 101 Commerce Street Newark, New Jersey 07102

Mr. David M. Scott, Acting Chief Bureau of Nuclear Engineering Department of Environmental Protection CN 411 Trenton, New Jersey 08625

## REQUEST FOR ADDITIONAL INFORMATION ON ATWS REVIEW RELATED TO ALTERNATE ROD INJECTION (ARI) AND RECIRCULATION PUMP TRIP (RPT) SYSTEMS OYSTER CREEK NUCLEAR GENERATING STATION

- . . .

Provide the following design information as supporting documentation of your method of compliance with 10 CFR 50.62

- 1. Please identify, by number in the NEDE-31096-P Appendix A checklist, any items which the station will not be in full conformance with the ARI SER.
- 2. Provide the electrical functional "agrams for the ARI and RPT systems from the sensors to the final actuated devices.
- 2.1 Describe the manual initiation capabilities and the protection against inadvertent operation.
- 2.2 Verify that no manual bypass of the RPT system is available.
- 2.3 Identify if there will be positive position indication for the ARI valves and the location of this indication.
- 3. Provide the electrical one-line diagram to demonstrate the electrical independence between the ARI system and the Reactor Trip System (RTS), and describe the capability to perform the ARI function during the loss -of-offsite power event.
- 3.1 Verify that the Reactor Recirculation Pump motor generator drive motor breakers have trip coils which are qualified. Verify both trip coils are independent of the plant safety systems. Identify the criteria applied to the design of these trip coils.
- Identify the separation criteria that is applied 3.2 between the RE02 and RE05 instrument loop signals which are located in the Foxboro cabinets. What separation criteria will be applied to the redundant ECCS chaniels and redundant RTS channels identified in item 6 of your submittal letter dated Sept 3,1987. Describe what is meant by the terms "adequate air space separation" and "a physical barrier (fire wrap) if necessary" in item 6 of your submittal. Identify the availability for staff review the analysis for all potential failure modes (assurance that no failure will affect the safety functions of both ARI and RPS) for the common power supply to sensors RE02 and RE05 and their instrument channels. Verify that item (f) of Appendix B to the SER of NEDE-31096-A has been considered and this information will be available for staff review.

3.3 Verify that the power source for the ARI control power, which is non-Class 1E, will be available after the loss of off-site power. Identify the ultimate source of power for the loss of off-site power condition.

- 1 - 2

- 4. Describe the system design which permits the test/calibration and maintenance of the system logic while the plant is in power operation.
- 5. The ARI actuation setpoints have been indicated as not challenging the RPS scram settings; identify the settings and levels to be used for both RPS and ARI.