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NUCLEAR ENGINEERING SERVICES DEPARTMENT  
CALVERT CLIFFS NUCLEAR POWER PLANT  
PACERSVILLE, MARYLAND 20657

March 21, 1988

U. S. Nuclear Regulatory Commission  
Washington, DC 20555

ATTENTION: Document Control Desk

SUBJECT: Calvert Cliffs Nuclear Power Plant  
Unit No. 1; Docket No. 50-317  
Partial Withdrawal of Proposed Technical  
Specification 3.1.1.1, Unit 1, (TAC NO. 67143)

REFERENCES: (a) Letter from Mr. J.A. Tiernan (BG&E), to Document Control Desk (NRC), Docket Nos. 50-317 and 50-318, "Request for Amendment-Unit 1 Cycle 10 License Application," dated February 12, 1988.

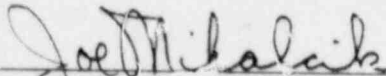
Gentlemen:

We understand that one of the proposed Technical Specification changes submitted in the Unit 1 Cycle 10 reload request for license amendment (Reference (a)), is currently under generic review by the NRC staff, pending a resolution in the future. This particular portion of the proposed change is not required for operation of Unit 1 Cycle 10, therefore we request to withdraw that portion.

Specifically, all changes proposed for page 3/4 1-1 of Technical Specification 3.1.1.1, should be withdrawn, except the reference to Figure 3.1-1b. The withdrawal of these proposed changes replaces applicability of Technical Specification 3.1.1.1 to the critical Modes of operation. By this withdrawal, the page number for Figure 3.1-1b will change from the proposed 3/4 1-2 to 3/4 1-2a.

The attached copies of Technical Specification 3.1.1.1 indicate the effect of this withdrawal on the proposed changes of Reference (a).

Certain changes to the Bases, as submitted in Reference (a), also result from this withdrawal action. These will be submitted via separate correspondence.

  
\_\_\_\_\_  
J. A. Mihalik  
Principal Engineer  
Fuel Cycle Management

JAM/DSE/lmt

Attachment

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cc: D. A. Brune, Esquire  
J. E. Silberg, Esquire  
R. A. Capra, NRC  
S. A. McNeil, NRC  
W. T. Russell, NRC  
D. C. Trimble, NRC  
T. Magette, DNR

### 3/4.1 REACTIVITY CONTROL SYSTEMS

#### 3/4.1.1 BORATION CONTROL

SHUTDOWN MARGIN -  $T_{avg} > 200^{\circ}F$

#### LIMITING CONDITION FOR OPERATION

3.1.1.1 The SHUTDOWN MARGIN shall be  ~~$\geq 3.5\% \Delta k/k$~~  <sup>equal to or greater than the limit line of Figure 3.1-1b.</sup>

APPLICABILITY: MODES 1, 2\*\*, 3 and 4.

#### ACTION:

<sup>less than the limit line of Figure 3.1-1b,</sup>  
With the SHUTDOWN MARGIN  ~~$\leftarrow 3.5\% \Delta k/k$~~ , immediately initiate and continue boration at  $\geq 40$  gpm of 2300 ppm boric acid solution or equivalent until the required SHUTDOWN MARGIN is restored.

#### SURVEILLANCE REQUIREMENTS

4.1.1.1.1 The SHUTDOWN MARGIN shall be determined to be  ~~$\geq 3.5\% \Delta k/k$~~  <sup>equal to or greater than the limit line of Figure 3.1-1b;</sup>

- a. Within one hour after detection of an inoperable CEA(s) and at least once per 12 hours thereafter while the CEA(s) is inoperable. If the inoperable CEA is immovable or untrippable, the above required SHUTDOWN MARGIN shall be increased by an amount at least equal to the withdrawn worth of the immovable or untrippable CEA(s).
- b. When in MODES 1 or 2<sup>#</sup>, at least once per 12 hours by verifying that CEA group withdrawal is within the Transient Insertion Limits of Specification 3.1.3.6.
- c. When in MODE 2<sup>##</sup>, within 4 hours prior to achieving reactor criticality by verifying that the predicted critical CEA position is within the limits of Specification 3.1.3.6.
- d. Prior to initial operation above 5% RATED THERMAL POWER after each fuel loading, by consideration of the factors of e below, with the CEA groups at the Transient Insertion Limits of Specification 3.1.3.6.

\* Adherence to Technical Specification 3.1.3.6 as specified in Surveillance Requirements 4.1.1.1.1 assures that there is sufficient available shutdown margin to match the shutdown margin requirements of the safety analyses.

\*\* See Special Test Exception 3.10.1.

# With  $K_{eff} \geq 1.0$ .

## With  $K_{eff} < 1.0$ .

## REACTIVITY CONTROL SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

e. When in MODES 3 or 4, at least once per 24 hours by consideration of the following factors:

1. Reactor coolant system boron concentration,
2. CEA position,
3. Reactor coolant system average temperature,
4. Fuel burnup based on gross thermal energy generation,
5. Xenon concentration, and
6. Samarium concentration.

4.1.1.1.2 The overall core reactivity balance shall be compared to predicted values to demonstrate agreement within  $\pm 1.0\% \Delta k/k$  at least once per 31 Effective Full Power Days (EFPD). This comparison shall consider at least those factors stated in Specification 4.1.1.1.1.e, above. The predicted reactivity values shall be adjusted (normalized) to correspond to the actual core conditions prior to exceeding a fuel burnup of 60 Effective Full Power Days after each fuel loading.

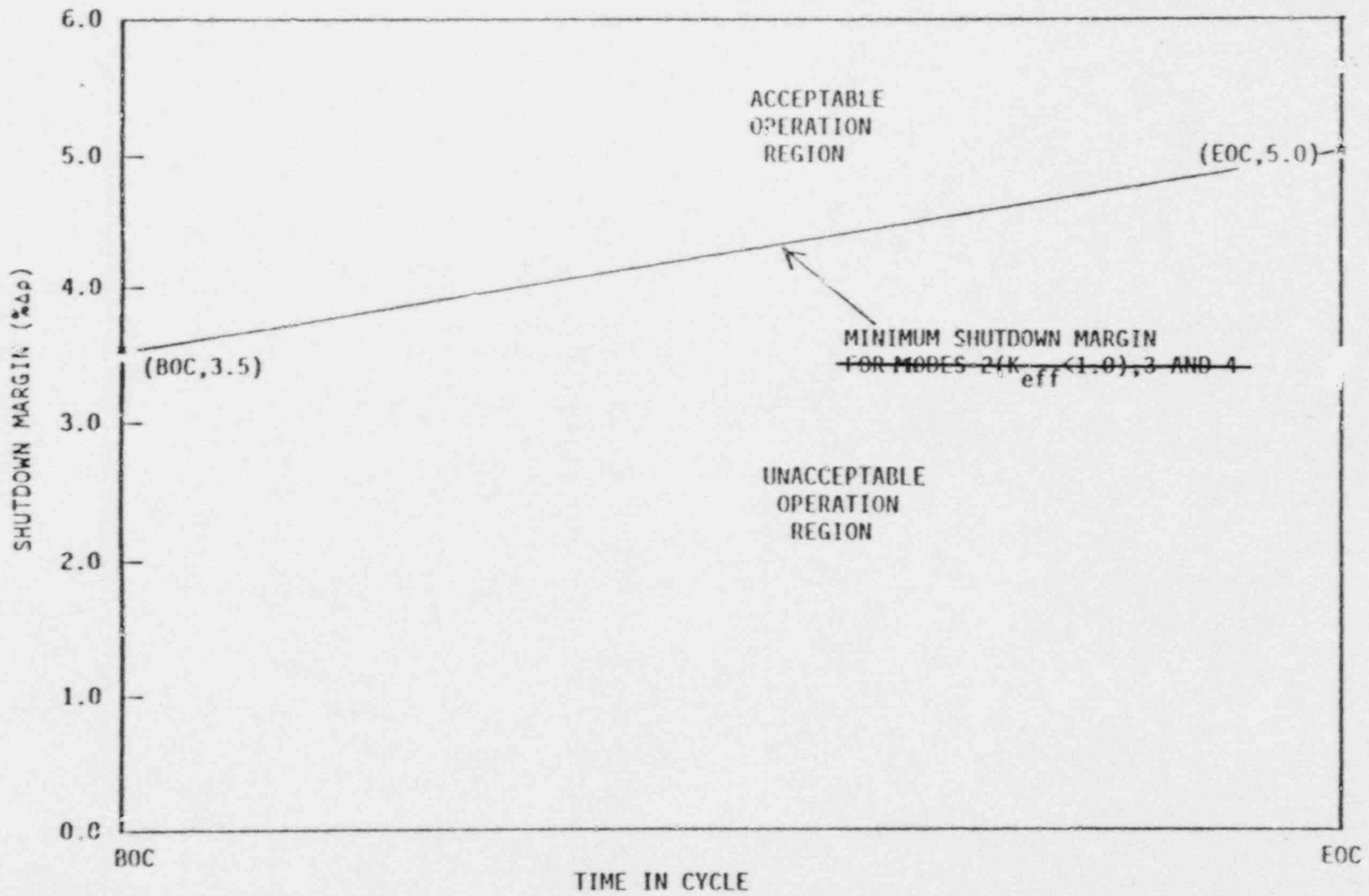


Figure 3.1-1b