

NOTICE OF VIOLATION

Commonwealth Edison Company

Docket Nos. 50-254; 50-265

As a result of the inspection conducted from September 14, 1987 through March 4, 1988, and in accordance with 10 CFR Part 2, Appendix C - General Statement of Policy and Procedure for NRC Enforcement Actions (1987), the following violations were identified:

1. 10 CFR 50, Appendix B, Criterion V, as implemented by CECO Topical Report CE-1-A, "Quality Assurance Program for Nuclear Generating Stations," and CECO Corporate Quality Assurance Manual, Nuclear Generating Stations, "Quality Requirements," requires that activities affecting quality shall be prescribed by documented drawings and shall be accomplished in accordance with these drawings.

Contrary to the above, certain activities were not accomplished in accordance with documented drawings in that:

- a. Structural steel connections inside the drywell were not accomplished in accordance with the General Electric drawings. Certain beam connections had missing welds, missing bolts or welds that were apparently cut so they were no longer effective. This caused ten connections to exceed the allowable design stress limits. (254/87028-01A)
- b. Support M-1610-18 and other supports, designed as part of the Mark I torus attached piping modifications, were not installed in accordance with the specified drawings. (254/87028-01B; 265/87028-03)

This is a Severity Level IV violation (Supplement 1).

2. 10 CFR 50, Appendix B, Criterion III, as implemented by CECO Topical Report CE-1-A, "Quality Assurance Program for Nuclear Generating Stations," and CECO Corporate Quality Assurance Manual, Nuclear Generating Stations, "Quality Requirements," requires that measures shall be established to assure that the design bases are correctly translated into drawings and that design control measures shall be applied to stress analyses.

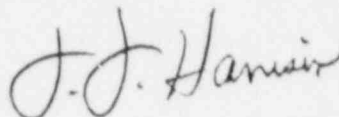
Contrary to the above:

- a. The fabrication drawings for embedment plates incorrectly specified an 18 inch spacing for anchor straps instead of 9 inches. The embedment plates were subsequently fabricated different from the design basis causing three plates to exceed the design stress limit. (254/87028-02A; 265/87028-02A)

- b. Design calculations for the Unit 1 Core Spray discharge line, Model Q1.10.2, contained incorrect pipe schedule information. (254/87028-02B; 265/87028-02B)

This is Severity Level IV violation (Supplement 1).

Pursuant to the provisions of 10 CFR 2.201, you are required to submit to this office within thirty days of the date of this Notice a written statement or explanation in reply, including for each violation: (1) corrective action taken and the results achieved; (2) corrective action to be taken to avoid further violations; and (3) the date when full compliance will be achieved. Consideration may be given to extending your response time for good cause shown.



MAR 18 1988

Dated _____

J. J. Harrison, Chief
Engineering Branch