# VIRGINIA ELECTRIC AND POWER CO (PANY RICHMOND, VIRGINIA 28261

November 2, 1978

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. O. D. Parr, Chief
Light Water Reactors Branch No. 3
Division of Project Management
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Serial No. 597 PO/FIT:scj Docket No. 50-338 License No. NPF-4

Dear Mr. Denton:

REQUEST FOR RELIEF FROM CERTAIN
REQUIREMENTS OF ASME XI FOR
INSERVICE TESTING OF PUMPS AND VALVES
FOR NORTH ANNA UNIT 1

### PUMPS

Our letter Serial No. 052D, dated September 29, 1977, proposed an inservice testing program for ASME Code Class 1, 2, and 3 pumps for North Anna Power Station, Unit 1. Our letter detailed the tests to be performed on each of the applicable pumps, and requested relief in several cases from the requirements of Section XI of the ASME Code. The NRC Staff, by letter dated October 17, 1977 concluded that our proposed program was acceptable for the first 20 month period following commercial operation of Unit 1, and granted relief from ASME Code requirements.

Our testing program, as outlined in our letter, Serial No. 52D, davad September 29, 1977, stated that flow instrumentation of the component cooling pumps 1-CC-P-1A, 1B and of the service water pumps 1-SW-P-1A, 1B was only accurate to +4% of full scale reading. Operating experience has shown that normal operating flows are approximately half or less than half of full scale flow. Thus accuracy of these gauges is at best +8% of normal operating flow. This accuracy does not lend itself to satisfying the requirements of Table IWP-2100-2, where the acceptable range of flow is +2%-6% of reference flow. In addition, varying the flow rates of these pumps to meet the reference flow interferes with normal plant operation since these flows have been balanced to meet the heat load requirements of the unit. Since these flow gauges were only designed to indicate relative flow and not to evaluate pump performance, an exception to the requirements of IWP-3100 is requested such that the flow and developed head of each pump will be recorded but will not be adjusted or compared to the reference value. The component cooling pumps are not required following an accident. Installation of more accurate

flow gaujes will be investigated during the first 20 months of commercial operation.

With the addition of the Casing Cooling System, two pumps should be added to the Attachment of our letter, Serial No. 052D dated September 29, 1977. These pumps are the Casing Cooling Pumps 1-RS-P-3A, 3B and will be tested per Subsection IWP to Section XI of the ASME Code - 1974 edition with addenda through Summer 1975.

The proposed changes to pump testing are attached and indicated by a vertical line in the right hand margin. Pages 1 and 15 supercede pages 1 and 15 and page 13b supplements page 13 of the attachment to our letter, Serial No. 052D, dated September 29, 1977.

### VALVES

Our letter Serial No. 052C, dated August 10, 1977, requested relief from certain requirements of ASME XI for inservice testing of valves for North Anna Unit 1. Included in this letter as an attachment was a statement of particulars and a tabulation listing all valves to be tested, their testing frequency and a statement of relief requested. The NRC staff, by letter dated October 17, 1977, concluded that our proposed program was acceptable for the first 20 month period follwing commercial operation of Unit 1, and granted relief from ASME Code requirements. Upon further review, we find it necessary to revise this attachment.

The proposed changes to valve testing as listed below are attached and indicated by a vertical line in the right hand margin. Pages 1 of Attachment C.O; 2, 7, 9, 10, 11 of Attachment C.I; and 5 of Attachment C.2 supercede the corresponding pages of the attachments to our letter, Serial No. 052C, dated August 10, 1977. Page 12 of Attachment C.1 and page 5a of Attachment C.2 supplement our August 10 letter.

- Delete part stroke testing requirement for the main steam trip valves in Attachment C. O. as part stroking of these valves could cause a unit trip. The valves will be tested for full closure time during each reactor shutdown but not more than once per 92 days.
- 2. MOV-1867A, B should be AMSE XI Cat. "B" rather than Cat. "A" and MOV-1867C, D should be Cat. "A" rather than Cat. "B". This was a typographical error.
- 3. Delete quarterly exercise testing requirements for MOV-1890A and B as explained in Note 27.
- 4. RHR relief valve, RV-1721, has been replaced by two relief valves, RV-1721A and 1721B. Include this change in attachment.
- 5. 1-SW-2 should be 1-SW-3. This was a typographical error.

- 6. Delete 1-FW-125, 1-FW-95, 1-FW-63 and 1-FW-61 which are auxiliary feedwater check valves. These valves are located immediately upstream of manual valves which are to be maintained closed and locked due to a design change. Therefore, these valves are not required to be tested.
- 7. Include 1-FW-279 which is a check valve in the discharge line from the steam driven auxiliary feedwater pump. Testing of this valve will be the same as is performed on other check valves in the auxiliary feedwater header.
- 8. Include MOV-RS-100A, B and MOV-RS-101A, B which are valves in the discharge line of the casing cooling pumps. These will be tested as ASME XI Category A valves except leakage testing will be performed in accordance with 10 CFR 50 APP. J in lieu of subsection IWV.
- 9. Delete 1-EG-272, 1-EG-284, 1-EG-289 and 1-EG-260 which are check valves in the discharge lines of Unit 2 emergency generator pumps and not required for Unit 1 operation.
- 10. Include 1-RS-146, 1-RS-147, 1-SI-312 and 1-SI-315 which are manual valves in the cross connection between recirc. spray and safety injection. These will be tested as ASME XI category E valves with no exceptions to testing requirements.
- 11. Include 1-RS-123 and 1-RS-138 which are check valves in the discharge line of the casing cooling pumps. These check valves shall be exercised during refueling outages as stated in Note 28 on Attachment C.2.

Very truly yours,

C. M. Stallings

Vice President-Power Supply and Production Operations

Attachments

cc: Mr. James P. O'Reilly

### COMPONENT COOLING PUMP

Class 3

1-CC-P-1A 1-CC-P-1B

MEASURED QUANTITIES

Pi - Monthly No exception\*

ΔP - Monthly\*\* Exception\*\*

Q - Monthly\*\* Exception to IWP - 4110 is required since

the available flow instrument is only 4%

accurate.\*\*

V - Monthly No exception

Proper Lubricant Level
or Pressure - Monthly No exception

Tb - Annually No exception

NOTES: These pumps are in a variable resistance system and are <u>not</u> required during an accident.

\*\* Values will be recorded monthly but not compared to reference values.

Additionally, motor current will be recorded for comparison purposes.

### CASING COOLING PUMPS

Class 3

1-RS-P-3A

1-kS-P-3B

## MEASURED QUANTITIES

Pi - Monthly No exception\*

ΔP - Monthly No exception\*

Q - Monthly\*\*\* See IWP-1400

V - Monthly No exception

Proper Lubricant Level

or Pressure

- Monthly No exception

Tb - Annually No exception

\*\*\*Not required but will be measured.

NOTE: The Casing Cooling Pump recirculation path is a fixed resistance

system.

SERVICE WATER PUMP

Class 3

1-SW-P-1A

1-SW-P-1B

MEASURED QUANTITIES

P: - Exception\*

ΔP - Monthly \*\*

0 - Monthly \*\*

Monthly

Proper Lubricant Level or Pressure

- Exception

T<sub>b</sub> - Not required

See below

Exception\*\*

Exception to IWP-4110 is required

since flow instruments are only

4% accurate. \*\*

No exception

See below

See IWP-4310

NOTES: The Service Water pump flow path is a variable resistance system.  $Proper\ Lubricant\ Pressure\ or\ Level\ can\ not\ be\ observed\ since$   $bearings\ are\ in\ the\ main\ flow\ path.\ Reference\ is\ made\ to\ IWP-4310$   $which\ establishes\ exception\ to\ T_b\ for\ bearings\ within\ the\ main\ flow$ 

path.

\*\* Values will be recorded monthly but not compared to reference values.

Additionally, motor current will be recorded for comparison purposes.

#### STATEMENT OF PARTICULARS

A review of class 1, 2, 3 valves has been completed for North Anna
Unit 1 Systems. Attachment C.1 provides a tabulation of the valves that
are subject to the testing requirements of ASME Boiler and Pressure Vessel Code,
1974 edition, subsection IWV with addenda through summer 1975. The table identifies the valves to be tested, valve code classes, and IWV category per IWV-2000.
Relief from the testing requirements of ASME XI is requested when they are
determined to be impractical. Specific information regarding the code requirement determined to be impractical and alternate testing programs are noted in
Attachment C.2

In addition to the valves listed in Attachment C.1, applicable containment isolation valves shall be leak tested and cycled at each refueling. North Anna Unit 1 Technical Specifications will list applicable valves and specify 10CF50 APP. J. Leak Testing will be conducted in accordance with APP. J in lieu of subsection IWV.

There are no testable Category D valves in North Anna Unit 1 Systems. All Category E valves shall be tested in accordance with IWV-3700.

Any inspection requirements identified as impractical during the course of the inspection period will be noted and included in the inspection program at the time of the next revision.

The main steam trip valves full closure time on any closure actuation signal will be verified while in HOT STANDBY during each reactor shutdown except that this verification need not be determined more often than once per 92 days for multiple shutdowns. In the event of continued unit operation, a shutdown will not be required for the sole purpose of performing the full closure test.

The residual heat removal system relief valves shall be tested whenever

				1	1		Page 2	
SYSTEM SI		ASME	ASME	RELIEF	TEST FREQUENCY		ASME XI CODE	
VALVE	VE FUNCTION		CAT.	REQUESTED	EXERCISE LEAKAGE		RELIEF REQUESTED	
MOV-1867 A, B	Boron Injection Tank Isolation Valves	II	В	NO	Every 3 Months	NA	NA	
MOV-1867 C, D	Boron Injection Tank Isolation Valves		А	NO	Every 3 Months (See Note 6)	Each Refueling	NA	
MOV-1869 A, B	High Head Safety Injection Off Charging Pump Header	II	AE	YES	Each Refueling	Each Refueling	NOTE 2	
MOV-1890 A, B	Low Head Safety Injection To Hot Legs	II	AE	yes	When in modes 4, 5, or 6 (See Note 27)	Each Refueling	Note 27	
MOV-1890 C, D	Low Head Safety Injection To Cold Legs	II	А	YES	Each Refueling	Each Refueling	NOTE 3	
MOV-1860 A, B	Low Head Safety Injection Pump Suction From Containment Sump	II	А	YES	Every 3 Months	NONE	NOTE 4	
1-SI-185, 79 201, 90	High Head Safety Injection Header Check Valves at Containment Penetrations	II	AC	YES	Each Refueling	Each Refueling	NOTE 5	
1-SI-206, 207, 197, 195, 199	Low Head Safety Injection Header Check Valves at Containment Fenetrations	II	AC	YES	Each Refueling	Each Refueling (NOTE 8 applies to 197, 195,199	NOTE 7	

RHR, QS	1	-				The state of the s
Min, go	ASME	ASME	RELIEF			ASME XI CODE RELIEF REQUESTED
ALVE FUNCTION		CAT.	REQUESTED	EXERCISE	LEAKAGE	REPIEL INVOISIED
Charging Pump Discharge Check Valve	11	С	NO	Every 3 Mont vs	NA	NV
Letdown Line Relief Valve Inside Containment	111	C	МО	NOTE 1	NA	NA
RHR Suction and Discharge from Reactor Coolant System	I	В	YES	Cold Shutdown	. NA	NOTE 16
RHR System Flow and Throttle Valve	II	В	NO	Every 3 Months	NA	NA
RHR Pump Discharge Check Valve	I	С	YES	Cold Shutdown	NA	NOTE 16
RHR System Relief Valve	II	С	NO	NOTE 26	NA	NA
Quench Spray Pump Discharge and Containment Isolation	II	Λ	NO	Every 3 Months	Each Refueling	NA
Quench Spray Pump Discharge and Containment Isolation Check Valve	11	AC	YES	Each Refueling	Each Refueling	NOTE 17
	Charging Pump Discharge Check Valve  Letdown Line Relief Valve Inside Containment  RHR Suction and Discharge from Reactor Coolant System  RHR System Flow and Throttle Valve  RHR Pump Discharge Check Valve  RHR System Relief Valve  Quench Spray Pump Discharge and Containment Isolation  Quench Spray Pump Discharge and	CAT.  Charging Pump Discharge Check Valve  Letdown Line Relief Valve Inside Containment  II  RHR Suction and Discharge from Reactor Coolant System  I  RHR System Flow and Throttle Valve  II  RHR Pump Discharge Check Valve  I  Quench Spray Pump Discharge and Containment Isolation  II  Quench Spray Pump Discharge and Containment Isolation  II  Quench Spray Pump Discharge and	CAT. CAT.  Charging Pump Discharge Check Valve  Letdown Line Relief Valve Inside Containment  II C  RHR Suction and Discharge from Reactor Coolant System  I B  RHR System Flow and Throttle Valve  II B  RHR Pump Discharge Check Valve  II C  RHR System Relief Valve  II C  Quench Spray Pump Discharge and Containment Isolation  II A  Quench Spray Pump Discharge and	CAT. CAT.  Charging Pump Discharge Check Valve  II C NO  Letdown Line Relief Valve Inside Containment  II C NO  RHR Suction and Discharge from Reactor Coolant System  I B YES  RHR System Flow and Throttle Valve  II B NO  RHR Pump Discharge Check Valve  II C YES  RHR System Relief Valve  II C NO  Quench Spray Pump Discharge and Containment Isolation  II A NO  Quench Spray Pump Discharge and	CAT. CAT.  CAT. CAT.  Charging Pump Discharge Check Valve  Letdown Line Relief Valve Inside Containment  II C NO NOTE 1  RHR Suction and Discharge from Reactor Coolant System  RHR System Flow and Throttle Valve  II B NO Every 3 Months  Every 3 Months  Cold Shutdown  RHR Pump Discharge Check Valve  II C YES Cold Shutdown  RHR System Relief Valve  II C NO NOTE 26  Quench Spray Pump Discharge and Containment Isolation  II A NO Every 3 Months  Every 3 Months  Every 3 Months	CAT. CAT.  EXERCISE LEARAGE  Charging Pump Discharge Check Valve  II C NO Every 3 Mont vg  NA  Letdown Line Relief Valve Inside Containment  II C NO NOTE 1 NA  MIR Suction and Discharge from Reactor Coolant System  I B YES Cold Shutdown  NA  RHR System Flow and Throttle Valve  II B NO Every 3 Months  NA  FHR Pump Discharge Check Valve  II C YES Cold Shutdown  NA  RHR System Relief Valve  II C YES Cold Shutdown  NA  RHR System Relief Valve  II C NO NOTE 26 NA  Quench Spray Pump Discharge and Containment Isolation  II A NO Every 3 Each Refueling  Quench Spray Pump Discharge and Containment Isolation  Quench Spray Pump Discharge and Each Each

		,	1	,	,		Page 9
SYSTEM SW. MS	FUNCTION		ASME XI CAT.	RELIEF REQUESTED	TEST FREQUENCY  EXERCISE LEAKAGE		ASME XI CODE PELIEF REQUESTED
1-SW-3, 10 2-SW-3, 10	Service Water Pumps Discharge Check Valve	III	С	NO	Every 3 Months	NA	NA .
1-SW-114, 116	Service Water to Recirc. Spray Heat Exchangers Check Valve	III	С	YES	Each Refueling	NA	NOTE 19
RV-SW-100A, B, C, D	Recirc. Spray Heat Exchanger Relief Valve	III	С	NO .	NOTE 1	NA	NA
TV-MS-101A, B, C	Main Steam Isolation Trip Valve	II	В	NO	Every 3 Months (see NOTE 21)	NA	. NA
TV-MS-111A, B	Steam to Turbine Auxiliary Feedwater Pump	II	В	NO	Every 3 Months	NA	ИЛ
1-MS-119, 122, 124	Main Steam to Turbine Auxiliary Feedwater Pump Check Valve	II	С	NO	Every 3 Months	NA	NA
102A, B, C 103A, B, C 104A, B, C	Main Steam Safety Valves	II	С	NO	NOTE 1	NA	NA
105A, B, C		BERTON STREET, AND ADDRESS OF THE STREET, STRE	and the second of the second o				

Page 10

rage 10										
SYSTEM FW, RS  VALVE			ASME XI CAT.	RELIEF REQUESTED	TEST FR	EQUENCY LEAKAGE	ASME XI CODE RELIEF REQUESTED			
HCV-FW-100A, B MOV-FW-100A, C	Auxiliary Feedwater Pump Discharge Isolation	III	Е	NO	NONE	NA	NA			
HCV-FW-100C MOV-FW-100B, D	Auxiliary Feedwater Pump Discharge Check Valve	III	В	YES	Each Refueling	NA	NOTE 3			
1-FW-47, 79, 111	Main Feedwater Check Valve at Containment Penetration	II	С	YŁS	Each Refueling	NA	NOTE 22			
1-FW-68, 100, 132 127, 93, 279	Auxiliary Feedwater Header Check Valves to Main Feedwater Header	III	С	YES	Each Refueling	NA	NOTE 23			
1-FW-148, 165, 183, 150, 167, 185	Auxiliary Feedwater Pump Discharge and Recirculation Check Valves	III	С	NO	Every 3 Months	NA	NA			
RV-FW-100	Turbine Auxiliary Feedwater Pump Discharge Relief Valve	III	С	NO	NOTE 1	NA	NA			
MOV-RS-156A, B	Recirc. Spray Pump Discharge and Containment Isolation	II	А	YES	Each Refueling	Each Refueling	NOTE 3			
MOV-RS-155A, B	Recirc. Spray Pump Suction from Containment Sump	II	А	YES	Each Refueling		NOTE 3, 4			

Page 11

rage II										
SYSTEM RS, GW, EG,			ASME XI CAT.	RELIEF REQUESTED	TEST FREQUENCY  EXERCISE LEAKAGE		ASME XI CODE RELIEF REQUESTED			
MOV-RS-100A, B MOV-RS-101A, B	Casing Cooling Pump Discharge to RS pumps.	II	А	NO	Every 3 months	Each Refueling	NA I			
1-RS-27, 18	Recirc. Spray Pump Discharge Check Valves at Containment Penetration	II	AC	YES	Each Refuelin	Each g Refueling	NOTE 17			
RV-GW-107A, B 100, 104A,B 103A,B,C,D 162A,B,C,D			С	YES	NOTE 24	NA	NA			
RV-EG-103A, B 104A, B 105A, B 106A, B	Emergency Generator Fuel Oil Pump Discharge Relief Valve	II	С	NO	NOTE 1	NA	NA			
1-EG-266, 295 254, 278	Emergency Fuel Oil Pump Discharge Check Valves	II	С	NO	Every 3 months	NA	NA			
MOV-CC-100A, B	Component Cooling Water to RHR Heat Exchanger and Containment Isolation	III	В	YES	Cold Shutdow	ı NA	NOTE 25			
1-CC-193, 198	Component Cooling Water to RHR Heat Exchanger and Containment Isolation	III	AC	YES	Cold Shutdow	Each Refueling	NOTE 25			
1-RS-146, 147 1-SI-312,315	Recirc. Spray Pump-Safety Injection Pump Cross Connection	II	E.	NO	NONE	NA	NA			

Page 12	ASME XI CODE	RELIEF REQUESTED	NOTE 28					
	TEST FREQUENCY	LEAKAGE						
	TEST FR	EXERCISE	Each Refueling					
		CEQUESTED	YES					
	ASME	CAT. CAT.	U				THE STATE OF THE S	
	ASME	CAT.	II			Table And Secretary College Co		
		FUNCTION	Casing Cooling Pump Check Valve to RS Pumps					
	SYSTEM RS	VALVE	1-RS-123, 138					

- integrity. This valve shall be exercised whenever the component cooling system for one unit is shutdown.
- 21. Refer to Attachment C.O which amplifies testing schedule.
- 22. To exercise this normally open check valve to the close position requires main feedwater flow to the steam generators to be secured and the initiation of auxiliary feedwater flow to back seat the disc. This valve shall be exercised when steam generator level and flow control is not required during a refueling outage.
- 23. It is impractical to exercise this check valve during power operation or cold shutdown per IWV-3520. The introduction of cold auxiliary feedwater to the steam generators induces unnecessary thermal stress on main feedwater piping systems. This check valve shall be exercised at refueling outages using auxiliary feedwater flow.
- 24. To prevent the release of radioactive contamination, relief valves on the waste gas decay tanks and gas stripper equipment shall be tested in accordance with IWV-3510 whenever these systems are out of service for maintenance and purged.
- 25. Exercising this valve would decrease flow of component cooling water to major operational components during power operation. It will be exercised during cold shutdown when flow is available to the RHR heat exchangers.
- 26. These relief valves cannot be tested unless the entire RHR system is not required for service. These valves will be tested when the system is out of service. The RHR system must be available at all times during operation and refueling outages for core cooling capability.

27. Technical Specification 4.5.2 requires these valves to remain closed with power to the operator removed while in MODES 1, 2, 3, and 4. These vlaves shall be exercised whenever the unit is in MODES 5 or 6 but not more than once per 92 days.

. . . .

28. It is impractical to exercise this check valve during power operation per 1WV-3520. Opening the test valve would break containment vacuum. The check valves shall be exercised at refueling outages.