

March 18, 1988

Docket No. 50-333

Mr. John C. Brons
Executive Vice President, Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

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Dear Mr. Brons:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - ATWS RULE (10CFR 50.62)
(TAC 59095)

Our letter dated January 8, 1987 requested that you provide plant-specific information concerning the adequacy of the FitzPatrick alternate rod insertion (ARI) system, the standby liquid control (SLC) system and the recirculation pump trip (RPT) system in meeting the requirements of the ATWS rule (10CFR 50.62).

Based on the information provided in your April 15, 1987 response, we have concluded that the SLC system design meets all applicable requirements of the rule. Our letter to you dated September 8, 1987 documented this finding.

Concerning the ARI system, we have identified additional information which will be needed in order to complete our review. This information is delineated in the enclosure and will require your response.

With regard to the RPT system, it is the staff's position that the original BWR/4 RPT design has not performed its function in a sufficiently reliable manner to comply with the ATWS Rule. Our January 8, 1987 letter requested that you provide a schedule for upgrading the existing system to either the Monticello design or the modified Hatch design, both of which were approved by the staff in 1979. As an alternative, you were requested to demonstrate that the existing system could perform its function with equivalent reliability.

Your April 15, 1987 response stated that you endorse the BWR Owner's Group position that redundant trip coils in each recirculation system motor-generator set, generator field breaker (as employed in the Monticello and Hatch designs) are not required to comply with the ATWS rule. Your letter further stated that "a submittal from the Owners Group to NRC regarding this matter is expected in the near future" and that "pending resolution of the issue, no modifications to the FitzPatrick RPT system are presently planned." To date, the NRC has not received a submittal from the Owners Group concerning this matter.

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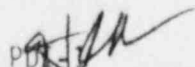
In the interest of obtaining a timely resolution of the ATWS Rule compliance issue for FitzPatrick, and in light of your commitment to complete all ATWS-related modifications prior to restart from the next refueling outage, scheduled to begin late in August, we request that you provide us with the information delineated above and in the enclosure, concerning the ARI and RPT systems.

Sincerely,

Harvey I. Abelson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosure:
As stated

cc: See next page


PDI
HAbelson:d1g
3/17/88

PDI-1
RCapra *Sum for*
3/18/88

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ENCLOSURE

REQUEST FOR ADDITIONAL INFORMATION CONCERNING
RECIRCULATION PUMP TRIP (RPT) AND
ALTERNATE ROD INJECTION (ARI) SYSTEMS FOR THE
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

Provide the following design information as supporting documentation to demonstrate compliance with 10CFR 50.62.

1. Provide electrical functional diagrams for the ARI and RPT systems from sensors to the final actuated devices. IS the ARI system separate from the existing RPT system? Is any portion common to both systems?
2. Provide the analysis results and test plans which verify that the ARI system design meets the requirements for timely completion. Also identify the ARI reset capabilities including the time delay to ensure ARI function has completed its operation.
3. Describe the ARI system manual initiation capabilities and the means provided to protect the system from inadvertent operation.
4. Identify the ARI/RPT information readout and indications provided in the control room. Will the ARI valve have a positive position indication?
5. Identify the interface between the ARI system and the safety related systems. Describe the Class 1E isolators which are used for this interfaces.
6. Describe the system design which permits maintenance and the test of the system logic while the plant is in power operation.
7. Describe the power source arrangement for the ARI and RPT systems, and the capability to function during the loss-of-offsite-power event.