



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 22, 1997

DECKET
FILE

50-282

Mr. Roger O. Anderson, Director
Licensing and Management Issues
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

SUBJECT: COMPLETION OF LICENSING ACTION FOR GENERIC LETTER 95-03,
"CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES," DATED
APRIL 28, 1995, FOR PRAIRIE ISLAND NUCLEAR GENERATING PLANT,
UNIT NOS. 1 AND 2 (TAC NOS. M92266 AND M92267)

Dear Mr. Anderson:

On April 28, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 95-03, "Circumferential Cracking of Steam Generator Tubes," to all holders of operating licenses or construction permits for pressurized-water reactors. The NRC issued GL 95-03 for three principal reasons:

- (1) Notify addressees about the safety significance of the recent steam generator tube inspection findings at Maine Yankee Atomic Power Station.
- (2) Request that all addressees implement the actions described within the generic letter.
- (3) Require that all addressees submit to the NRC a written response regarding implementation of the requested actions.

In addition, GL 95-03 alerted addressees to the importance of performing comprehensive examinations of steam generator tubes using techniques and equipment capable of reliably detecting the types of degradation to which the steam generator tubes may be susceptible. The staff also noted that the performance of steam generator tube examinations is controlled, in part, by Appendix B to Title 10, Part 50, of the *Code of Federal Regulations* (10 CFR Part 50).

In GL 95-03, the NRC staff also requested that licensees take the following actions:

- (1) Evaluate recent operating experience with respect to the detection and sizing of circumferential indications to determine the applicability to their plants.
- (2) On the basis of the evaluation in Item (1) above, as well as past inspection scope and results, susceptibility to circumferential cracking, threshold of detection, expected or inferred crack growth rates, and other relevant factors, develop a safety assessment justifying continued operation until the next scheduled steam generator tube inspections.

NRC FILE CENTER COPY

DF01/0

9705300095 970522
PDR ADOCK 05000282
P PDR

- (3) Develop plans for the next steam generator tube inspections as they pertain to the detection of circumferential cracking. The inspection plans should address, but not be limited to, scope (including sample expansion criteria, if applicable), methods, equipment, and criteria (including personnel training and qualification).

To document the outcome of these actions, the NRC staff requested that addressees prepare and submit the following:

- (1) A safety assessment justifying continued operation, predicated on the evaluation performed in accordance with requested actions 1 and 2 (above).
- (2) A summary of the inspection plans developed in accordance with requested action 3 (above) and a schedule for the next planned inspection.

In response to GL 95-03, you provided letters dated June 27, 1995, and April 8, 1996, for Prairie Island Nuclear Generating Plant Units 1 and 2. These submittals provided the information requested by GL 95-03; therefore TAC Nos. M92266 and M92267 are closed. For your information, the staff's findings regarding this issue are contained in NUREG-1604, "Circumferential Cracking of Steam Generator Tubes."

If you have any questions regarding this matter, please contact me at (301) 415-1355.

Sincerely,

Original signed by

Beth A. Wetzel, Project Manager
 Project Directorate III-1
 Division of Reactor Projects - III/IV
 Office of Nuclear Reactor Regulation

Docket Nos: 50-282 and 50-306

cc: See next page

DISTRIBUTION

Docket File

PUBLIC

PD# 3-1 Reading

J. Roe

OGC

ACRS

K. Karwoski

S. Dembek

J. McCormick-Barger, RIII

DOCUMENT NAME: G:\WPDOCS\PRAIRIE\PI92266.LTR

To receive a copy of this document, indicate in the box C = Copy w/o attachment/enclosure E = Copy with attachment/enclosure N = No copy

OFFICE	PM:PD31	E	LA:PD31	E	D:PD31	
NAME	BWetzel:db		CJamerson		JHannon	
DATE	5/21/97		5/21/97		5/21/97	

OFFICIAL RECORD COPY

Mr. Roger O. Anderson, Director
Northern States Power Company

Prairie Island Nuclear Generating
Plant

cc:

J. E. Silberg, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington DC 20037

Site Licensing
Prairie Island Nuclear Generating
Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, Minnesota 55089

Plant Manager
Prairie Island Nuclear Generating
Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, Minnesota 55089

Tribal Council
Prairie Island Indian Community
ATTN: Environmental Department
5636 Sturgeon Lake Road
Welch, Minnesota 55089

Adonis A. Neblett
Assistant Attorney General
Office of the Attorney General
455 Minnesota Street
Suite 900
St. Paul, Minnesota 55101-2127

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
1719 Wakonade Drive East
Welch, Minnesota 55089-9642

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, Illinois 60532-4351

Mr. Jeff Cole, Auditor/Treasurer
Goodhue County Courthouse
Box 408
Red Wing, Minnesota 55066-0408

Kris Sanda, Commissioner
Department of Public Service
121 Seventh Place East
Suite 200
St. Paul, Minnesota 55101-2145