GPU Nuclear

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U.S Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
10 CFR 50.62 (c)(4)

The NRC Staff, by letter dated February 18, 1988, requested that GPU Nuclear submit a Technical Specification Change Request (TSCR) that will provide limiting conditions for operation and surveillance requirements for the Standby Liquid Control System by March 31, 1988. The purpose of this letter is to inform the Staff that the requested submittal date does not allow sufficient time for the preparation and approval of the TSCR and that GPUN intends to submit the information by May 1, 1988.

Very truly yours,

Michael Laggart FOR

J. R. Thorpe
Director, Licensing & Regulatory Affairs

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cc: Mr. William T. Russell, Administrator
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NRC Resident Inspector
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Mr. Alex Dromerick, Jr.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FEB 18 1988

Docket No. 50-219

Mr. P. B. Fiedler
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: STANDBY LIQUID CONTROL SYSTEM COMPLIANCE WITH THE ATWS RULE - 10 CFR 50.62(c)(4) - OYSTER CREEK NUCLEAR GENERATING STATION (TAC NO. 66126)

By letter dated September 3, 1987 and December 30, 1987, GPU Nuclear Corporation provided information concerning the requirements of 10 CFR 50.62 for Oyster Creek Nuclear Generating Station. Based on our review of this information, we have concluded that the proposed design for the Standby Liquid Control System is consistent with the requirement of 10 CFR 50.62(c)(4) and is acceptable. This is discussed in the enclosed Safety Evaluation. Other matters related to the requirements of 10 CFR 50.62 will be the subject of a future letter from the staff when the evaluation is completed.

It is requested that you submit a Technical Specification Change Request that will provide limiting conditions for operations and surveillance requirements for the Standby Liquid Control System as discussed in the staff's Safety Evaluation. You are requested to respond to this request by March 31, 1988 so that the staff can complete its review in accordance with the schedule completion date.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 97-511.

Sincerely,

Alexander W. Dromerick

Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure
See next page

Mr. P. B. Fiedler
Oyster Creek Nuclear Generating Station

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ENCLOSURE 1

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATING TO 10 CFR 50.62(c)(4), ATWS RULE
GPU NUCLEAR CORPORATION
OYSTER CREEK NUCLEAR GENERATING STATION
DOCKET NO. 50-219

1.0 INTRODUCTION

In letters dated September 3, 1987 (5000-87-1361) and December 30, 1987 (5000-87-1447) GPU Nuclear Corporation submitted a description of its plans for implementing the requirements of 10 CFR 50.62 at Oyster Creek Nuclear Generating Station. The proposed Standby Liquid Control System (SLCS) is designed to supply 30 gpm of enriched sodium pentaborate solution (35 atom percent of B-10) to the reactor vessel to meet ATWS requirements.

2.0 EVALUATION

The SLCS description provided by GPU for Oyster Creek has been reviewed by the staff against the requirements of the ATWS rule (10 CFR 50.62(c)(4), and Generic Letter 85-03, "Clarification of Equivalent Control Capacity for Standby Liquid Control Systems," dated January 28, 1985. The licensee will increase the boron enrichment to a minimum of 35 atom percent of B-10, and proposes to supply 30 gpm of a minimum 15 weight percent of sodium pentaborate solution to the reactor vessel. Accounting for the physical size of the Oyster Creek reactor vessel which is 213 inches inside diameter, the aforementioned flow/enrichment combination satisfies the ATWS Rule equivalency requirement which is based upon 86 gpm pump flow, 13 weight percent sodium pentaborate and 19.8 atom percent B-10, and a 251 inch diameter vessel, as discussed in Generic Letter 85-03. The proposed design is therefore acceptable.

The licensee should confirm that surveillance and positive verification will be performed periodically (once per month) to assure that the correct isotopic enrichment is maintained. When additional chemicals, e.g., boron or boric acid are added to the storage tank, isotopic enrichment of B-10 in the solution should also be verified. This commitment as well as periodic verification of the 30 gpm flow rate should be reflected in the plant Technical Specifications.

3.0 CONCLUSIONS

The description of the proposed SLCS operation by GPU Nuclear Corporation for the Oyster Creek Nuclear Generating Station in letters dated September 3, 1987 and December 30, 1987 is acceptable because it is consistent with the equivalency requirements of 10 CFR 50.62(c)(4).

Principle Contributor: M. McCoy