



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION IV
611 RYAN PLAZA DRIVE, SUITE 1000
ARLINGTON, TEXAS 76011

MAR 4 1988

Docket No. 50-458
License No. NPF-47
EA 87-229

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President, (RBNG)
Nuclear Licensing
Post Office Box 220
St. Francisville, LA 70775

Gentlemen:

SUBJECT: NOTICE OF VIOLATION (NRC INSPECTION REPORT NO. 50-458/87-28)

This refers to the inspection conducted during the period November 1-21, 1987, of activities at the River Bend Station by personnel of the NRC Region IV office. During this inspection, NRC personnel reviewed the circumstances associated with a violation of NRC requirements involving a misaligned instrument root valve which was identified by the licensee. This violation was discussed in detail at an enforcement conference December 14, 1987, at the NRC Region IV office in Arlington, Texas in which you and your staff provided further details regarding the systems and components affected by the misaligned instrument root valve, the sequence of events leading up to your identification of the problem, analysis of the safety significance, and corrective actions you have implemented or plan to implement.

Violation A in the enclosed Notice of Violation (Notice) involves the failure to maintain the minimum number of channels operable as required by Technical Specifications for reactor trip system initiation, high pressure core spray (HPCS) system initiation, and various system isolation initiation. This was caused when personnel failed to open an instrument root valve for a drywell pressure sensing line which served three pressure transmitters. This valve remained closed from August 1985 to November 17, 1987. With this valve closed, various automatic safety functions initiated by a high drywell pressure signal (i.e., reactor scram, HPCS initiative, and system isolation) were unavailable for short periods of time while required periodic testing was performed and at other times channel redundancy was lost for these various safety functions. The NRC recognizes that other system initiation signals were unaffected and that alarms and indications existed in the control room which could have alerted personnel to a high drywell pressure condition. We also note that this violation and Violation B of the enclosed Notice, while separated by a period of two years, both involved the failure of operations management to ensure that procedures were followed in the performance of valve lineups.

In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1987), Violation A described in the enclosed Notice has been classified at a Severity Level III. A civil penalty is considered for a Severity Level III violation. However, after

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consultation with the Director, Office of Enforcement, and the Deputy Executive Director for Regional Operations, I have decided that a civil penalty will not be proposed in this case because of your prompt identification and reporting of the violation and the unusually prompt and extensive corrective actions, including a detailed evaluation. However, further similar incidents may result in civil penalty action.

While we view this Severity Level III violation as a serious problem requiring strong management action, we also recognize that this problem was identified because of your self-initiated action to verify safety-related valve lineups during this refueling outage. Without this self-initiated action, this problem may have gone undetected for another refueling cycle or longer. Also, we note that this problem was identified by plant personnel and immediately brought to management and NRC attention and you have encouraged this climate by formally commending the operators who discovered the problem, a practice which the NRC encourages from all licensees.

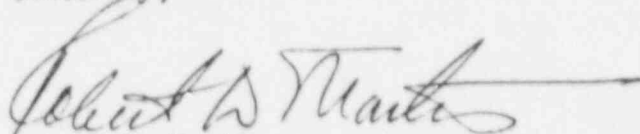
Two violations (B and C) are also described in the enclosed Notice and have been categorized as Severity Level IV violations. These violations involved the failure to follow procedures while doing an inservice reactor pressure vessel leakage test and for processing an extension for a modification request.

You are required to respond to this letter and should follow the instructions specified in the enclosed Notice when preparing your response. In your response, you should document the specific actions taken and any additional actions you plan to prevent recurrence. After reviewing your response to this Notice, including your proposed corrective actions and the results of future inspections, the NRC will determine whether further NRC enforcement action is necessary to ensure compliance with NRC regulatory requirements.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and its enclosure will be placed in the NRC Public Document Room.

The responses directed by this letter and the enclosed Notice are not subject to the clearance procedures of the Office of Management and Budget as required by the Paperwork Reduction Act of 1980, Pub. L. No. 96-511.

Sincerely,



Robert D. Martin
Regional Administrator

Enclosure:
Notice of Violation

cc:
Louisiana Radiation Control Program Director