

April 7, 1997

LICENSEE: Niagara Mohawk Power Corporation

FACILITY: Nine Mile Point Nuclear Station Unit No. 1

SUBJECT: SUMMARY OF TELEPHONE CONVERSATION OF APRIL 3, 1997, REGARDING  
RECIRCULATION PIPING WELD

On April 3, 1997, the NRC staff participated in a telephone conference call with Niagara Mohawk Power Corporation (NMPC and licensee) to discuss the licensee's observations from inspections of Weld 32-WD-050 between the recirculation system suction piping and isolation valve 32-376 at Nine Mile Point Nuclear Station, Unit 1. The list of participants is given in Enclosure 1. Enclosure 2, which was telecopied to the NRC for the telephone conference, summarizes the licensee's discussion. The licensee's evaluation of the flaw is in progress.

The flaw was observed by ultrasonic testing (UT) to be about 1/4 inch in depth relative to the piping inner diameter and located about 3 inches from the long (axial) seam of the piping in a root repair area of the weld that was made in 1982/1983 when the recirculation piping was replaced. The licensee plans to recheck (1) the Induction Heating Stress Improvement (IHSI) record for the weld from the 1986 inservice inspection to confirm that the stress relief technique was properly applied and (2) the post IHSI automated raw UT data for the presence of a flaw.

The licensee believes that the indication is either a reflector caused by an unsuccessful repair at installation or cracking initiated due to stresses in the localized repair area. Based on Generic Letter 88-01, the licensee is considering maintaining the weld classification as Intergranular Stress Corrosion Cracking Category A, even though NMPC intends to increase the frequency of inspection at least during the next two refueling outages. The NRC staff strongly recommended that the licensee consider reclassification as Category F, which requires reinspection each refueling outage.

Sincerely,  
/s/

Darl S. Hood, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-220

- Enclosures: 1. List of Participants
- 2. Telecopied Text and Sketches

cc w/encls: See next page

DOCUMENT NAME: G:\NMPI\RECIRC.TEL

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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2. Telecopied Text and Sketches

cc w/encls: See next page

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April 3, 1997

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J. Swenszkowski  
J. Cilento  
et al.

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W. Koo  
W. Lazarus (RI)  
D. Hood

Enclosure 1

Post-It™ brand fax transmittal memo 7871		# of pages > 4
To D. Hood	From A. Zaffino	
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Dept. Pers. Mgr	Phone # 315-377-7428	
Fax # 301-415-2602	Fax # 315-377-1400	

## RECIRCULATION SYSTEM WELD INDICATION RESPONSES TO NRC QUESTIONS

### 1. Previous preservice (PT, UT, RT) and inservice findings:

**ET:** 1983 PSI - (11) rounded indications, all less than 1/16" in diameter. 1997 ISI - (1) 1/4" rounded indication recorded.

**UT:** 1983 PSI - UT identified root geometry and counterbore geometry 360° at various amplitudes. 1986 post IHSI - automated UT identified "no relevant indications found."

**RT:** 1983 root pass info - lack of fusion noted between 17"-19" location markers - repaired, re-shot and accepted. Final ASME Section III RT accepted on 1-25-83. It should be noted that the present UT indication is at the same location as the root repair area discussed above. 1997 supplemental RT - performed supplemental RT on this indication this outage. Indication was not seen with conventional or enhanced digitization of the RT film. Installation and supplemental RT film was digitized, however, the areas of interest did not positively reveal this indication.

### 2. Indication of a crack? Does it show IGSCC characteristics?

- Indication is a non-geometric planar reflector. There is not enough information to positively characterize this indication at this time.
- The indication appears to be clearly contained within the root repair area with 45° shear, 60° R.L., and WSY-70. The apparent tip signals observed with IGSCC sizing techniques were low amplitude.
- This indication does exhibit some IGSCC characteristics such as location and skewing.

### 3. Any fracture mechanics status or results?

NMPC has contracted MPR Associates to perform an analytical evaluation of the indication per the 1986 Edition of ASME Code Section XI. The evaluation is currently underway. Preliminary results show that the flaw size calculated for the end of the upcoming operating cycle meets the Table IWB-3641 allowable flaw size criteria. The

evaluation considers flaw growth, stress corrosion cracking as well as by fatigue. These results are preliminary and have not been reviewed or accepted by NMPC.

4. **Confirm the weld was a pipe field replacement weld, the piping material specification and the pipe wall thickness.**

Weld 32-WD-050 was replaced during the outage between 3/19/82 and 6/4/83. The root weld was installed on 12/21/82 and final welding completed on 1/11/83. Root radiography report was rejected because of an area of incomplete fusion, which was repaired. No further repairs were made to the weld.

Materials:

Valve 32-376 is A351 Gr CF8M

28" Piping is SA 358 Cl. 1 Gr 316, Modified 0.02% Max Carbon

Weld Metal is ER-308L 0.035" and IN-308L 1/8" x 5/32"

Pipe Wall Thickness: Specified Min. Wall 1.050"; UT Measurement 1.1".

5. **Radiation levels?**

1,000 mr on contact (system drained); 500 mr at 1 foot.

6. **Any unusual history that might be tied to a cause?**

See response to question 4. NMPC has verified that area of incomplete fusion identified and repaired in 1983 (installation) is in the same location as indication found in RFO14. UT performed in 1986 following IHSI identified "no relevant indications found."

7. **Overall conclusion on indication.**

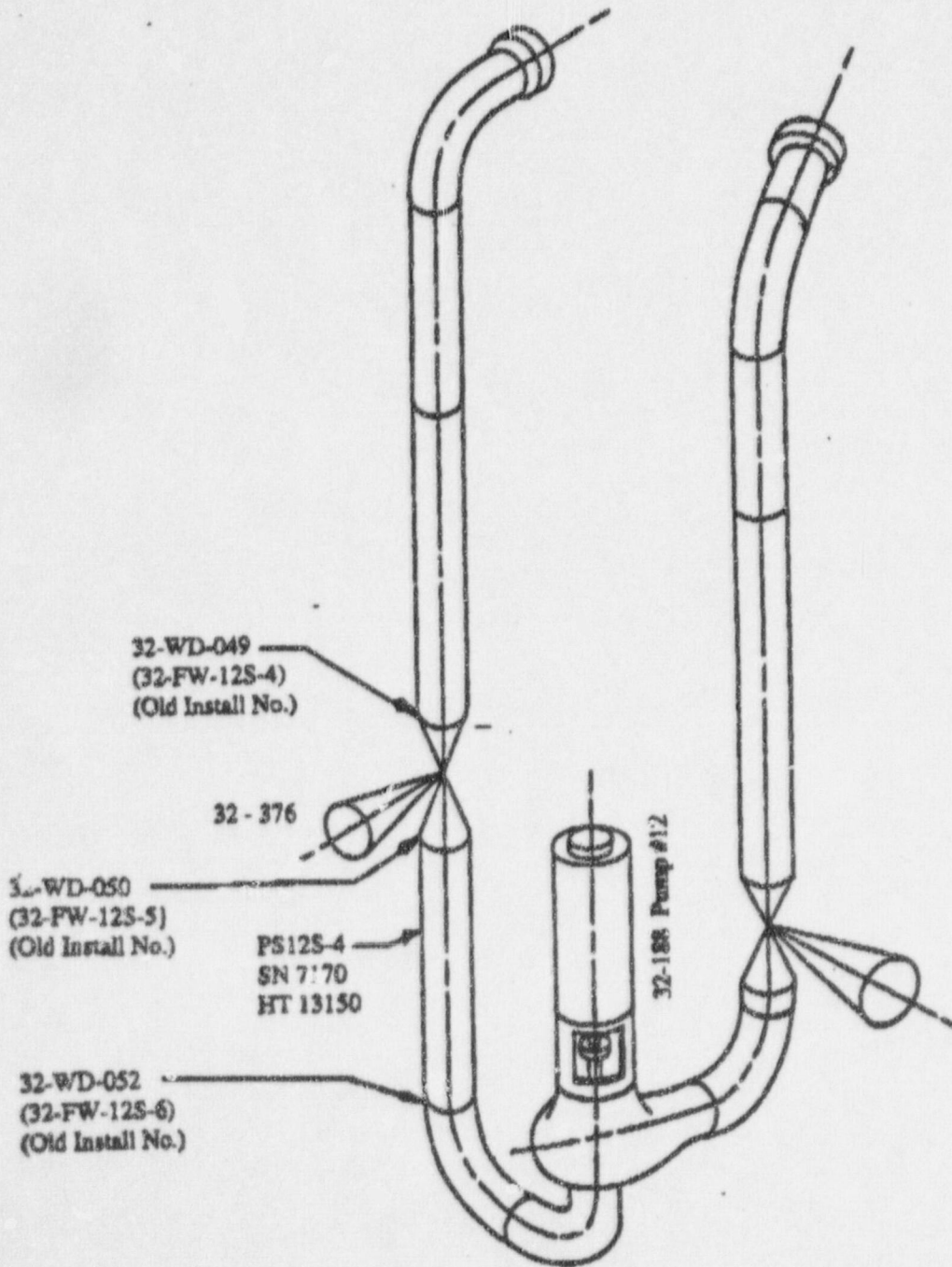
Indication is either a reflector caused by an unsuccessful repair at installation or cracking initiated due to stresses in the localized repair area.

**ADDITIONAL INFORMATION****Location:**

See simplified sketch. Upstream of weld 32-WD-050 is the loop 12 suction blocking valve 32-376 and downstream is a 11'-5.5" length of 28" pipe.

**Expanded Examination:**

An expanded sample of 9 additional examinations on the recirculation system were chosen. All additional examinations have been completed and found to be acceptable. Also, approximately 30% of the welds in the recirculation system have been UT examined since piping replacement with no rejectable indications found.



NMP1  
Loop 12  
Reactor Recirculation





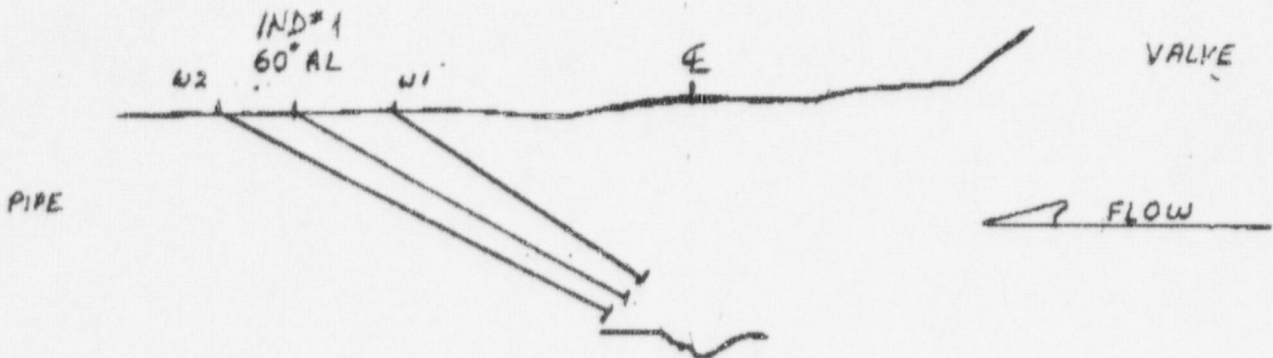
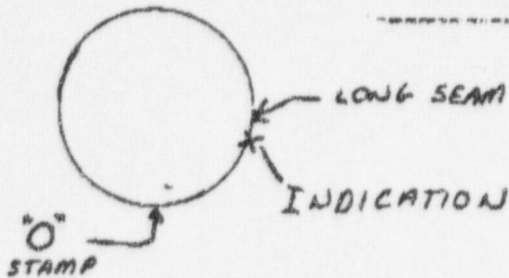
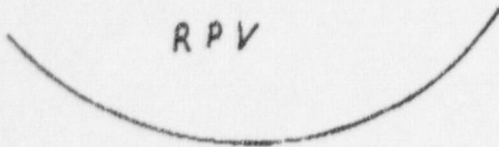
SKETCH SHEET

NDE Report: 1-6.03-97-0219  
 Work Document: NMP-151-002 R.O  
 Page 3 of 4  
 Exam Item: 32-WD-050  
 Procedure/Rev.: 6.03 REV-11

Plant/Unit: Mine Mile Point Unit 1  
 ISO/Dwg.: F-45183-C SH.7 REV'S  
 System: 32.0

Post-it™ brand fax transmittal memo 7671 # of pages 1

To <u>D. Wood</u>	From <u>A. Zelnick</u>
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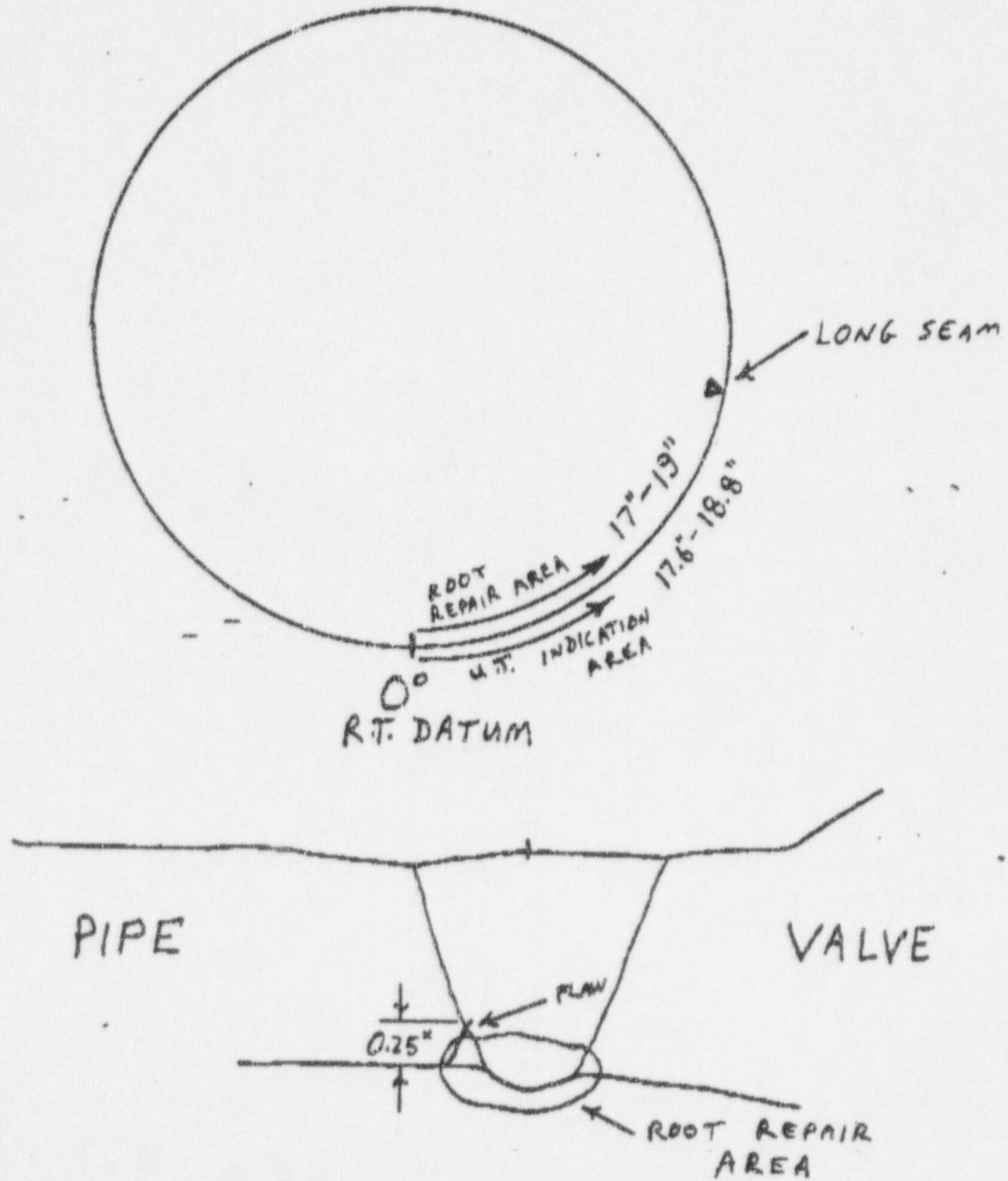


NOTE: PEAK AMPLITUDE ATTAINED BY SKEWING TRANSDUCER  
 10"-15" BACK TOWARD "0"

Examiner 1: Walter Holloway Level: II Company: G.E. Date: 3-18-97  
 Examiner 2: N/A Level: N/A Company: N/A Date: N/A  
 Reviewer 1: James R. Miles Level: II Company: NIC Date: 3/22/97  
 ANII: \_\_\_\_\_ Date: \_\_\_\_\_

Plant/Unit: Nine Mile Point Unit 1  
 ISO/Dwg.: F-45183-C  
 System: 32.0 RX RECIRC

NDE Report: INFORMATION SKETCH: FLAW LOCATION  
 Work Document: RELATIVE TO ROOT  
 Page 1 of 1 REPAIR AREA  
 Exam Item: WELD # 32-WD-050  
 Procedure/Rev: N/A



Examiner 1: J.J. CILENTO Level: III Company: NMPC Date: 4-3-97

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To: <u>D. Hood</u>	From: <u>A. Zallnick</u>
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