

March 2, 1988

Docket No. 50-368

Mr. T. Gene Campbell  
Vice President, Nuclear  
Operations  
Arkansas Power and Light Company  
Post Office Box 551  
Little Rock, Arkansas 72203

Dear Mr. Campbell:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION, REACTOR COOLANT PUMP TRIP,  
ARKANSAS NUCLEAR ONE, UNIT 2 (TAC. NO. 49676)

After reviewing your response (November 24, 1986, 2CAN118608) to Generic Letter 86-06, Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps", we find that we need additional information so that we may complete our review. Please provide answers to the enclosed questions within 50 days of the date of this letter.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

15/

George F. Dick, Jr., Project Manager  
Project Directorate - IV  
Division of Reactor Project - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosure:  
As stated

cc w/enclosure:  
See next page

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Arkansas Power & Light Company

Arkansas Nuclear One, Unit 2

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Honorable William Abernathy  
County Judge of Pope County  
Pope County Courthouse  
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ADDITIONAL QUESTIONS ON  
ARKANSAS POWER AND LIGHT'S RESPONSE TO  
GENERIC LETTER 86-06  
FOR  
ARKANSAS NUCLEAR ONE, UNIT 2

1. Arkansas Power & Light's (AP&L's) letter of November 24, 1986 did not clearly identify which of the criteria presented in CEN-268 was selected to trip the second set of pumps during a small break LOCA (SBLOCA) at Arkansas Nuclear One, Unit 2 (ANO-2). Identify the criterion selected and the setpoints used to determine when to trip the second set of pumps. Also, identify the pressure setpoint used to trip the first set of pumps if different from that recommended in CEN-268.
2. For the instrumentation identified in response to question 1, discuss how the effects of instrument uncertainty, as identified in AP&L's November 24, 1986 response to Generic Letter (GL 86-06 item 2) were included in determining the setpoints for pressure, subcooled margin, and secondary reactivity.
3. AP&L did not provide sufficient information in its November 24, 1986 response to GL 86-06 item 3 to determine how the uncertainties in the generic analysis presented in CEN-268 affect the results as they apply to ANO-2. Therefore, identify the ANO-2 plant specific features not representative of the reference plant used in the analyses presented in CEN-268. At a minimum discuss core power; decay heat; HPIS capacity; makeup flows; setpoints for reactor trip, safety injection, and accumulator injection and show that the values used in the generic analysis are either representative of those at ANO-2 or conservative. If a reference plant parameter is not representative for ANO-2, discuss how this was considered in determining the plant specific setpoints.
4. Part 2 of Item 4 of Generic Letter 86-06 requested the licensee to identify, "...procedures which provide direction for use of individual steam generators with and without operating RCPs." In its November 24, 1986 response, AP&L stated that because all four RCPs are not tripped for transients that cause use of individual steam generators and the first two pumps tripped are always in opposite loops, use of individual steam generators has no relationship to RCP trip criteria. Because the ANO-2 procedures will always allow one pump in a loop to be operating in situations where a single steam generator will be used, identifying all procedures and operator training that involve use of single steam generators is sufficient to comply with this item. The licensee is requested to provide this information.

5. In its response to item 4, AP&L stated that emergency operating procedure (EOP) 2202.01 requires the use of reactor coolant pump trip guidelines. Identify what situations, i.e., main steam line breaks, steam generator tube ruptures, small break LOCAs, etc., are covered by EOP 2202.01.